

Data for Modeling the Irradiation of Mixed Oxide Fuel in the SAXTON Reactor

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Fissile Materials Disposition Program

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1. Introduction

1.1. Goal

With the end of the cold war, thousands of nuclear warheads have been deemed excess to national security needs by both the United States and Russia [1]. The inter-governmental U.S./Russia Joint Study on Plutonium Disposition has determined that one of the means by which weapons-plutonium could reach the secure spent fuel standard is that it be processed as fuel in a light water reactor. Historically, the United States has not been involved in the use of Plutonium in reactors since 1976 while Russia has had very little experience with plutonium in light water reactors. In order to fulfill this new disposition mission however, there is a need to use past data to benchmark neutronics codes utilized nowadays in nuclear power plants operations. The object of this report is to analyze the data from the Saxton Plutonium Program performed by Westinghouse under sponsorship of the U.S. Government in the 60's and early 70's. This report will present these data and compare them with results obtained with a two-dimensional neutronics collision probability code called HELIOS [40].

Historically, the Saxton critical experiments [15] have been regarded as one of the only U.S. data available involving lattice moderated MOX rods. Hence these experiments have been the starting point, worldwide, for the evaluation and benchmarking of MOX lattices in foreign reactors (see for instance [2]). These critical experiments were a small set of a much larger program called the Saxton Plutonium Program which lasted ten years (1964-1974) and saw the first use of mixed oxide fuel in a power generating reactor. This small set of critical experiments were performed at Waltz Mill, PA in mid-1965 while the Saxton Plutonium Program was undertaken at the Saxton reactor in Pennsylvania. During this undertaking, most critical experiment configurations did not have more than 250 fuel rods and hence had very large bucklings and leakages. These small critical experiments were studied in a previous publication [3]. Our report aims at describing operations of the much larger Saxton cores I, II and III described in the different Westinghouse quarterly progress reports to AEC/DOE and NRC [5-39]. We have attempted to implement these core configurations with the HELIOS computer code system (version 1.4). HELIOS is a collision probability neutron and gamma transport code system for lattice burn-up calculations in general two-dimensional geometry. Unfortunately, due to lack of funding, computational studies could not be completed.

The primary goal of reviewing data from an old project such as the Saxton Plutonium Program is to try to find a good reference on the performance of plutonium-loaded cores. The information presented in this report is expected to provide a benchmark for recently available nuclear data libraries and computer code systems. An aspect of our computations includes a comparison with similar computer generated data used during the project. Since our approach would use a two dimensional code, and, since the current state-of-the-art in transport computation does not allow for rapid, three-dimensional transport solvers, it is expected that the output of HELIOS will provide, at least, cross sections valuable for three-dimensional analyses using diffusion codes.

1.2. History

The purpose of the Saxton Plutonium Program was to develop information concerning the utilization of plutonium enriched fuel in pressurized water reactor systems, through design, fabrication and operation of a partial core of $\text{PuO}_2\text{-UO}_2$ fuel in the Saxton reactor. In-pile performance of this fuel was evaluated and post irradiation examination of fuel samples was made. The plutonium core configuration used as a basis for the analysis consisted of nine plutonium fuel assemblies installed in the center of the core with twelve uranium fuel assemblies installed on the periphery. Seven of the nine plutonium fuel assemblies contained pelletized fuel while two contained vibratory compacted (VIPAC) fuel. In order to provide a reference to uranium based cores, the designers made Core I a uranium-only core. Cores II and III were uranium-plutonium loaded cores. The three cores operated at or near full power during most of the program period. Detailed information about this experimental program is open and available in the form of quarterly reports which cover almost the whole program. [5-39].

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2. System description

2.1. Reactor core overview

The Saxton reactor was originally designed to hold 32 fuel assemblies. However, only 21 fuel assemblies were used. Some details about Saxton reactor Core I and Core II are summarized below:

	Core I UO₂		Core II UO₂ PuO₂ - UO₂	
Number of fuel assemblies	UO ₂ – 21		UO ₂ – 12 PuO ₂ – 9	
Total number of assemblies	21		21	
Number of rods per assembly	UO ₂ – 71		UO ₂ – 71	
Equivalent core diameter	71.2978cm	28.07 in	71.2978cm	28.07 inches
Core length	92.964 cm	36.6 in	92.964 cm	36.6 inches
Measured weight of UO₂ in the core	1023.3043 kg	2256 lbs	-	

Number of rod positions available :

21 assemblies of 72 rods each	1512
6 followers of 18 rods each	108
9 inserts of 9 rods each	81
Total	1701
23 instrumentation thimbles	23
2 secondary source tubes	2
Net number of rods for potential use	1676

Please note that the number of rod positions available reveals a potential for utilization. While there are 72 rod positions available in every assemblies, only 71 were used in either core I, II and III. A typical fuel assembly is presented in Figure 1. Core I was a uranium-only core. The numbering scheme for the location of every assembly is given in Figure 2.

Generic reactor core characteristics:

- Thermal power, MW thermal: 20 (Core I) and 23.5 (Core II)
- Moderator temperature, K: 549.8167 (Core I) and 551.4830 (Core II)
- Operating pressure, MPa (psi): 13.7895 (2000)

Special characteristics of the Saxton Core II configuration:

Core II occurred shortly after Core I. The configuration of Core II is similar to that of Core I except for the fact that the nine central UO₂ assemblies were replaced with mixed oxide (PuO₂ - UO₂) assemblies.

Ten of the eleven peripheral UO_2 assemblies were replaced with fresh ones (all assemblies except for location E-1 - see Fig. 2). Out of the nine mixed oxide assemblies, seven assemblies contained pelletized fuel and the remaining two assemblies (D-2 and D-4) contained VIPAC fuel. The basic difference between pelletized and VIPAC fuel resides in the theoretical density: the density of VIPAC fuel is less than that of the pelletized fuel. Most of the mixed oxide fuel rods were clad in zircalloy-4 and a few (around 30) were clad in type 304-SS. The stainless steel clad rods were distributed throughout the core, for comparative purposes.

Special characteristics of Saxton Core III configuration:

The general objectives of the investigations, which were carried out with the Saxton Core III configuration, were the following:

- Demonstration of the performance capability of MOX fuel elements with Zircaloy-4 cladding over a broad spectrum of fuel burnups and reactor power levels, achievable in power reactors;
- Validate predictions of fuel element performance, including determination of power/burnup failure limitations;
- Obtain depletion characteristics for high burnup MOX fuel.

The construction of the nine center assemblies and of the unirradiated peripheral UO_2 assemblies remains essentially the same as that of Cores I and II, except that these assemblies have been reinforced. Two locations in each of the twenty-one main fuel assemblies in the core (A-5 and E-1) were used for either flux thimbles, removable fuel rods, removable cladding test specimen assemblies, removable water-filled tubes containing spot-weld test specimens or removable solid zircaloy bars. The nine central assemblies differ from previous assemblies in that the top nozzle is removable.

Each top nozzle is fastened on to the assembly by three stainless steel tie-rods and held captive by nuts with integral locking cup washers. The tie-rods consisted of type 304-SS tubing of 0.99314cm (0.391”) diameter with a 0.0381cm (0.015”) wall thickness, and type 304-SS special end plugs. The bottom end plug of each tie rod is welded into the bottom nozzle and the rod passes through the grids in the same manner as a fuel rod. The top end plug consists of two square sections with a threaded section at the top end.

2.2. Fuel assembly

Each fuel assembly was a 9x9 configuration (see Fig. 1). Out of the eighty-one positions available, seventy-one were occupied by fuel rods and one location was earmarked for the instrumentation thimble.

In the case of the nine central assemblies, the remaining nine rod locations located along an edge were slotted to form an L-shaped profile to accommodate the control rods (see Fig. 4). In the case of the peripheral assemblies, these L-shaped sections contained uranium fuel at these locations. The L-shaped section inserts were made of type 348-SS with a thickness of 0.07112cm (0.028”). Five assemblies out of the twenty-one were annular in configuration in the sense that they had provisions for insertion of experimental 3x3 or 2x2 subassemblies. Their locations were D-3, E-1, B-3, F-4 and C-5 in Figure 2. The naming convention for these locations became increasingly sophisticated during the course of the program. In Figure 3, the same locations were named N-1, N-4, N-2, N-5 and N-3 respectively and were named as such in the references.

The basic components of a fuel assembly consisted of four grids, two enclosure halves and one each top and bottom nozzles. The grid assemblies were of brazed “egg-crate” construction and were spaced axially at 25.4 cm spans (ten-inch spans) to provide lateral support for the fuel rods. The enclosure halves were welded to the peripheral straps of the grid assemblies to support the grids and to tie the fuel assemblies together. The nozzles were welded to the top and bottom ends of the enclosure halves.

Special characteristics of the Saxton Core II configuration:

The peripheral uranium oxide fueled assemblies had a U-235 enrichment of 5.7 wt. %. The central mixed oxide fueled assemblies had natural uranium oxide and 6.6 wt % plutonium oxide fuel, with an isotopic enrichment of 90.49 % Pu-239, 8.57 % Pu-240, 0.89 %Pu-241 and 0.04 % Pu-242.

The removable subassembly in the central location D-3 was previously irradiated in Core I to burnup levels of 1400 MWD/t. In contrast to the rest of the fuel assembly in D-3, the inter-element pitch in this sub-assembly was 1.36652cm (0.538”). Assembly in location E-1 contained a special, hollow, 51 UO₂ rod configuration, designed to accommodate a supercritical steam test loop. This assembly had already accumulated 2100 MWD/t burnup during Core I operation. Out of the 12 UO₂ assemblies, five were original of Core I design (with SS grids) and seven were new, Core II design (Inconel grids). All UO₂ assemblies, except for E-1, were fresh fuel at the start of Core II. All the mixed oxide (PuO₂ - UO₂) assemblies contained inconel grids.

The special L-shaped insert assemblies that occupied the slotted regions of the fuel elements were also used in Core II. In addition, the original, fuel-bearing control rod followers were also used in Core II. However, the control rods 1 & 2 and 5 & 6 were interchanged, placing the fuel followers with the lowest burnups at the center of the core.

The central, PuO₂-UO₂-fuelled assembly (D-3) and three peripheral (B-3, F-4, C-5) UO₂-fuelled assemblies had provisions for the insertion of 3x3 subassemblies.

Special characteristics of the Saxton Core III configuration:

The 21 main fuel assemblies (9x9 rod array) in Core III were made up of seven loose lattice assemblies, two special fuel test (load-follow) assemblies, one peripheral non-irradiated UO₂ assembly and eleven UO₂ assemblies from Core I (see Fig. 2).

The seven loose lattice assemblies consisted of four 36-rod assemblies, one 35-rod assembly, one 34-rod assembly and one 32-rod assembly. This last assembly accommodated, in addition to 32 fuel rods, a four rod removable subassembly. The rods in these loose lattice assemblies were composed of irradiated rods removed from Core II assemblies. They were loaded into the original 9x9 assemblies with zircaloy tubes filled with coolant at every other rod position. As in Core II, in each of the main fuel assemblies, two rod positions were used for either in-core instrumentation, source rods or removable fuel rods, depending on the location of the fuel assembly in the core. In addition, the L-shaped assemblies in the corner of the peripheral assemblies were reused with fuel assemblies in Core III.

Each of the two special fuel test (load-follow) assemblies C-3 and E-3 contained sixty fuel rods with design variations in fuel pellet diameter, density, and internal gas pressure. The unirradiated peripheral UO₂ assembly contained pelletized, enriched UO₂ and zircaloy-4 clad fuel rods.

The remaining fuel loading consisted of six control rod followers already in the reactor, four removable 3x3 type fuel assemblies in the peripheral UO₂ assemblies and the L-shaped assemblies in the peripheral slot positions already in place in Core II.

Summary of fuel assemblies in the Saxton reactor Core III:

Description:	Number of Assemblies			
	72-Rod	63-Rod	36 /35 /34-Rod	32-Rod
Loose lattice assemblies, MOX	-	-	6	1
Special test assemblies, UO ₂	2	-	-	-
Cores I & II , UO ₂ assemblies	8	4	-	-

Construction and loading plan for the loose-lattice assemblies

The loose-lattice assemblies were loaded with a loose lattice pitch of 2.0828cm (0.82”). A water tube occupied each alternate rod position on the normal lattice pitch of 1.3732cm (0.58”). The water tubes, as well as the tie rods, had two holes, 0.15875cm (1/16”) diameter, near the bottom, and two holes, 0.3175cm (1/8”) diameter, at the top of the cladding, to permit the flow of primary coolant through the rods. To maintain uniform fuel rod pitch across the core, two types of fuel assemblies were necessary and were called Type A and Type B assemblies. The 32-rod Type A assembly contained, in addition to the 32 rods, a removable subassembly with four fuel rods arranged with a

1.92532cm (0.758") pitch. These four fuel rods in this 3x3 subassembly consisted of the mixed oxide rods with the highest burnups from previous operation in Core II. This assembly was designed as a removable assembly so that the rods could be removed to monitor core performance.

The fuel rods in the 9x9 loose lattice assemblies were selected from previously irradiated Core II assemblies. The loose lattice plutonium region is made up of seven reconstituted assemblies that were subdivided into zones distinguished by the amount of rod burnup in Core II. At intermediate locations between the loose lattice fuel rods, open zircalloy tubes were installed to provide additional moderator while maintaining the desired flow characteristics. Three of these intermediate positions contained open SS tubes that were used to attach the fuel assembly nozzle.

The fuel assemblies were strengthened by replacing the six zircaloy-4 tubes with six SS tubes and by spot welding 0.07112cm (0.028") thick SS angles between the can and the six SS water-filled tubes. The 2.54cm (one inch) long clips, which previously fastened the two halves of the enclosure were replaced by full-length clips in the spans between the grids. The loose-lattice region of the core contained one L-shaped assembly at core location E-2. This L-shaped assembly contained nine pelletized UO₂ fuel rods clad with type 348-SS.

Construction and loading plan for the load-follow assemblies

At the beginning of Core III life, core locations E-3 and C-3 were occupied by load-follow assemblies 503-18-3 and 503-18-1 respectively. However, at mid-life, these locations were reversed. These two assemblies contain 60 fuel rods arranged with a nominal pitch of 1.4732cm (0.58"). They have the same dimensions as Core II type fuel rods and contain pelletized UO₂ fuel with two enrichments (9.5 or 12.5% weight percent (w/o) U-235). The pellet densities range between 89.5 and 94.5% of theoretical density and have nominal pellet-to-clad gaps of 0.01397cm to 0.02413cm (5.5 to 9.5 mils).

Each of the assemblies also contained four water-filled zircaloy-4 tubes and one water-filled SS tube. The tie-rods in each load follow assembly contained filler material. Two contained inconel and one contained SS fillers. The purpose of the filler material was to control local power perturbations in the assemblies. The load follow assemblies were also structurally strengthened.

The removable rod position at location A-5 was occupied by a solid zircaloy bar. The removable rod position at location E-1 was occupied by a flux thimble tube. Assembly 503-18-1 was located in C-3. The cladding test specimen was located in assembly 503-18-3 at core location E-3. Locations A-5 and E-1 were blank (not much information on location A-5 is given in the series of reports).

Other assemblies

Eleven previously irradiated assemblies from Core I were loaded into eleven of twelve Core III peripheral locations. The other peripheral location, B-2, was occupied by a non-irradiated enriched UO₂ assembly designated 503-10-7. This assembly contained 68 zircaloy-4 clad fuel rods containing pelletized UO₂ fuel arranged on a 1.4732cm (0.58") pitch. This assembly was also strengthened similar to the load-follow assemblies. However, this assembly contained solid zircaloy-4 bars at both the E-1 and A-5 removable rod locations; whereas the load follow assemblies contained only one solid zircaloy-4 bar.

Subassembly design

The removable subassemblies could be placed at several positions in the core as shown in Fig.3. They were similar in construction to the main fuel assemblies except that the can was made of 0.04826 cm (0.019") thick perforated SS and the rods were arranged on a 1.36144cm (0.536") pitch (see Fig. 4).

Subassembly 503-4-31 was initially in the N-1^a location. This subassembly contained four previously irradiated mixed oxide fuel rods clad in zircaloy-4, a flux thimble in the center rod location and four water filled zircaloy-4 tubes in the four corner locations.

Subassembly 503-4-33 was initially in the N-2^a location. This subassembly contained two hydriding effects test fuel rods which contained fuel having average D₂O concentration of 120 ppm and two high-pressure creep rods at 13.203 MPa (1915 Psi) initial pressure. This subassembly also contained a flux thimble in the center location and four water-filled zircaloy-4 tubes at the four corner locations.

Subassembly 503-4-25 was initially in the N-3^a location. This subassembly contained five non-removable fuel rods clad with type 304-SS and enriched to 5.7 % w/o U-235. In addition, this subassembly contained four removable fuel rods clad with zircaloy-4, enriched to 5.7 % w/o U-235, with various internal pressures.

Subassembly 503-4-32 was initially in the N-4^a location. This subassembly contained four non-removable fuel rods clad with zircaloy-4, enriched to 12.5 % w/o U-235; two low-pressure creep test rods; a flux thimble, and two irradiated zircaloy-4 clad, PuO₂-UO₂ rods.

Subassembly 503-4-34 was initially in the N-5^a location. This subassembly contained two hydriding effects test fuel rods and two materials compatibility test rod.

a- See Fig.3

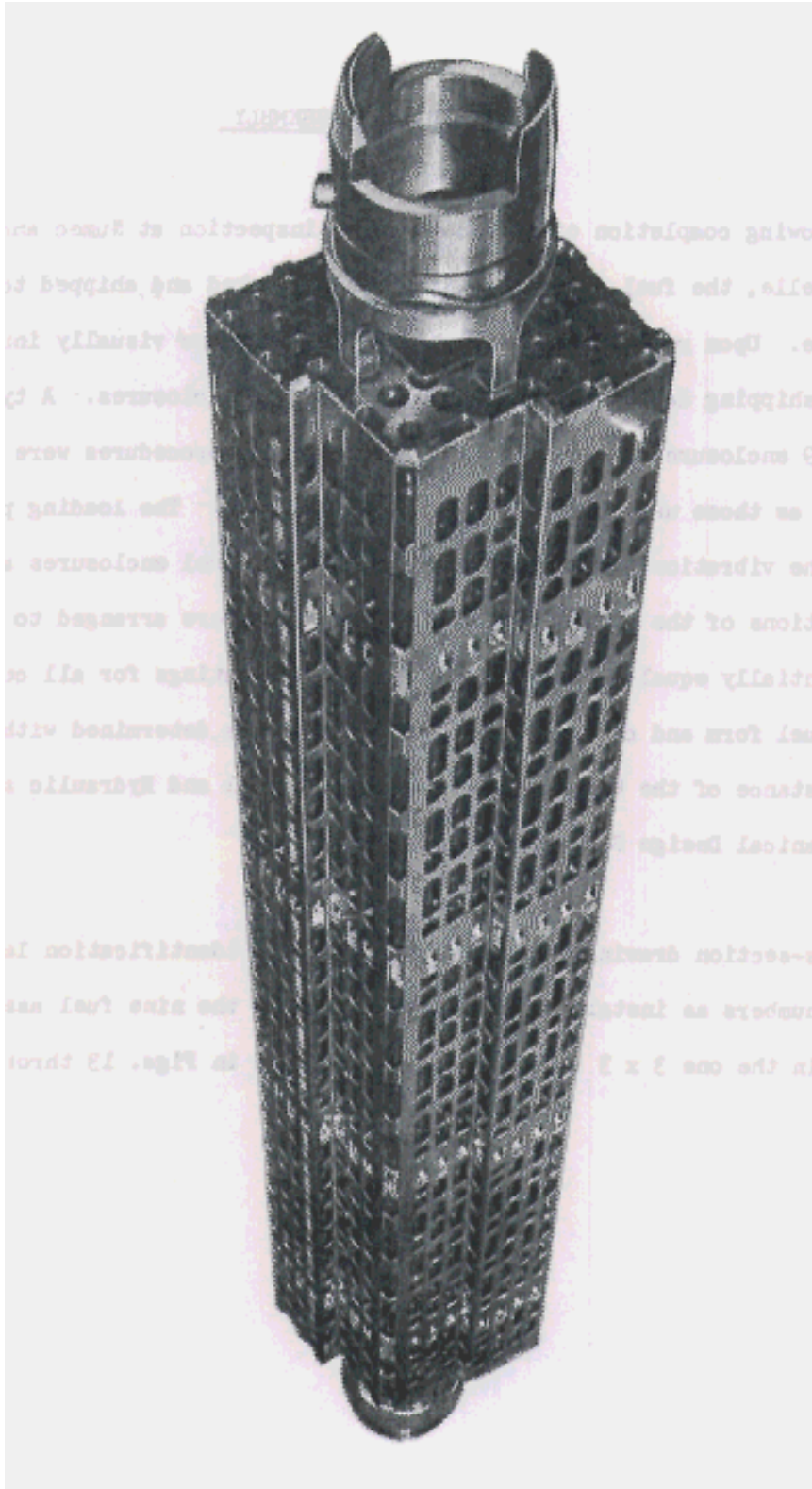
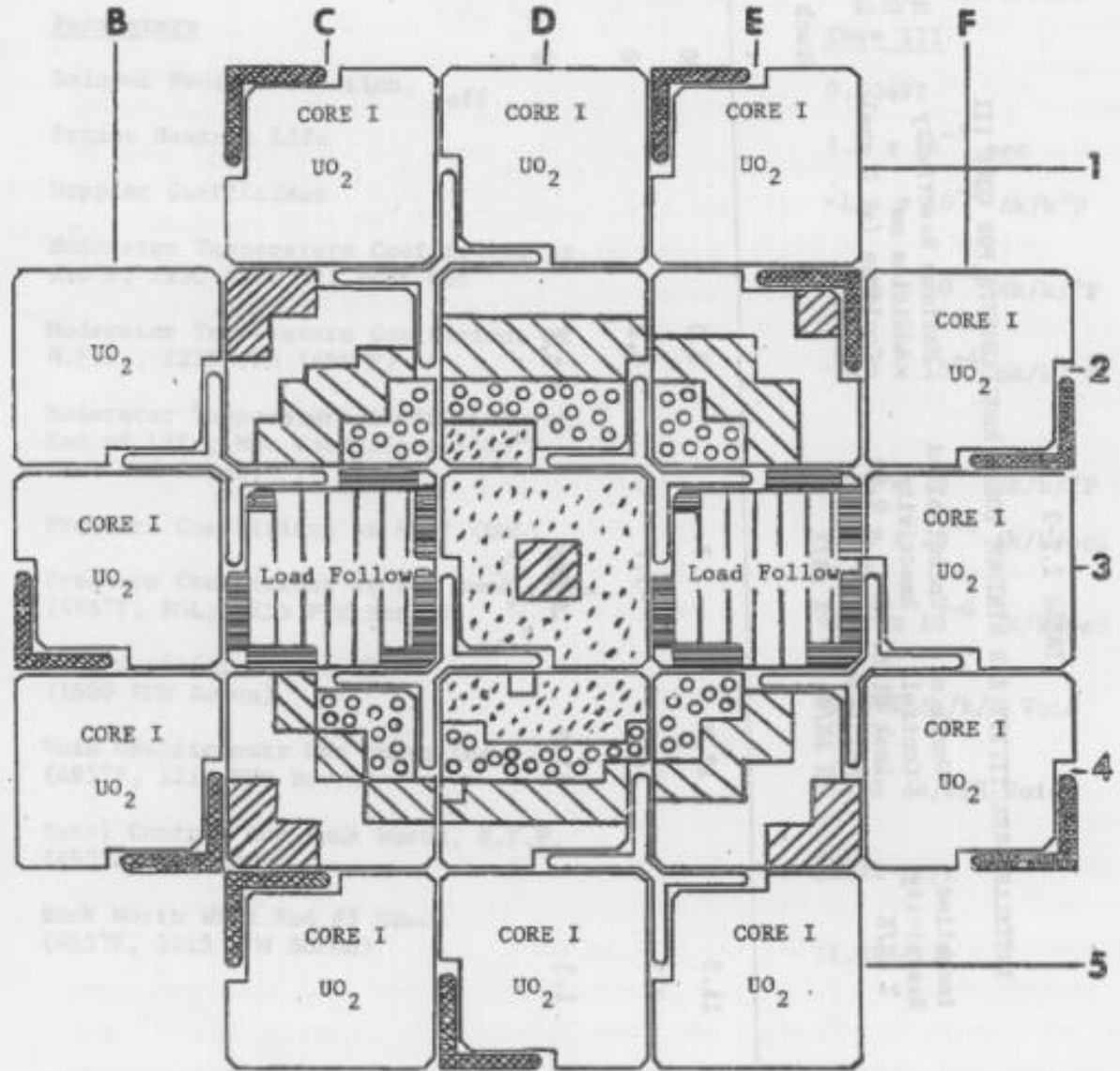


Figure 1: Saxton fuel assembly

BURNUPS AND ENRICHMENTS IN LOAD-FOLLOW AND LOOSE-LATTICE ASSEMBLIES
 USED IN CORE III NUCLEAR DESIGN



Loose-Lattice Assemblies
 C-2, D-2, E-2, D-3, C-4, D-4, E-4
 Rod Average Burnup from Core II

- ▨ 13,500 MWD/MTM
- ▩ 15,000 MWD/MTM
- ▧ 17,000 MWD/MTM
- ▦ 19,000 MWD/MTM
- ▤ 23,000 MWD/MTM

Load-Follow Assemblies
 UO₂ Enrichment

- ▨ 9.5 w/o U-235
- ▩ 12.5 w/o U-235

Figure 2: Core III map

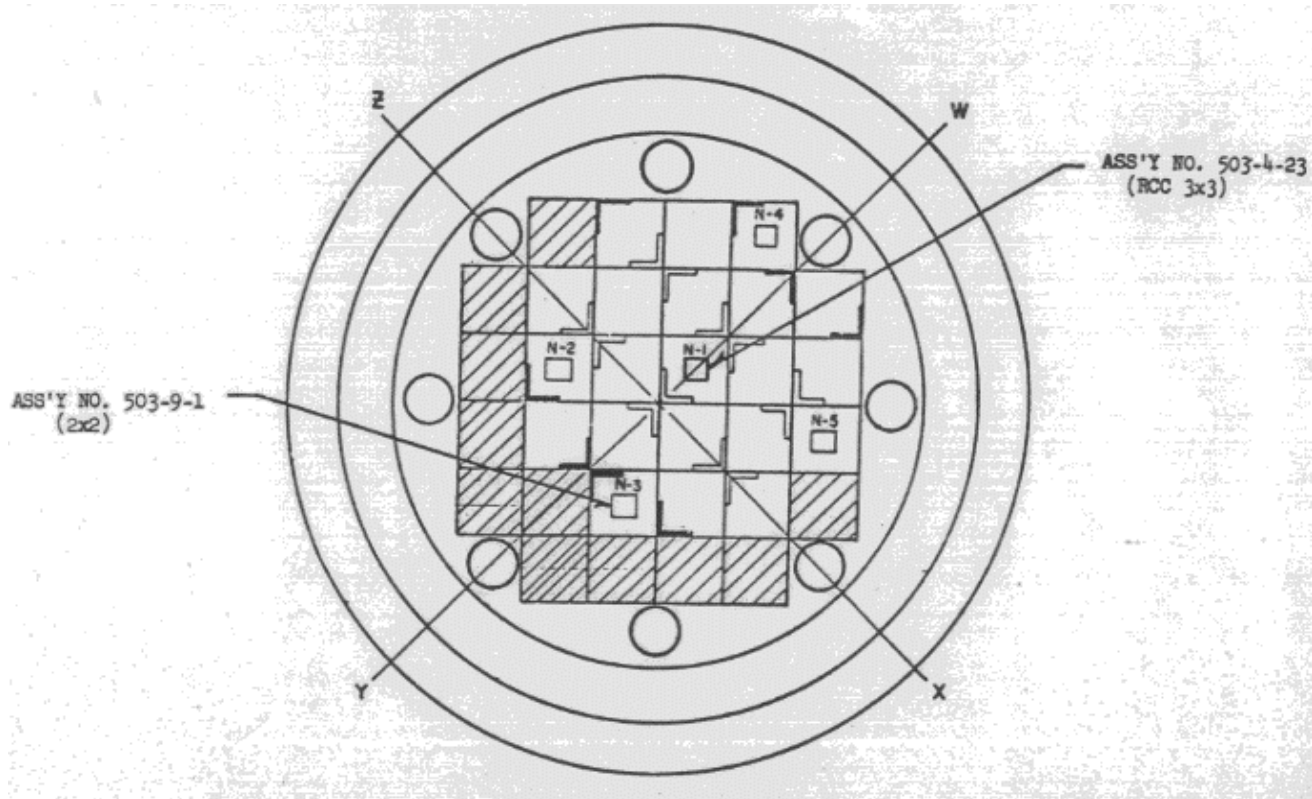


Figure 3: Global map of the Saxton core and location of the special removable 3x3 (RCC) and 2x2 assembly

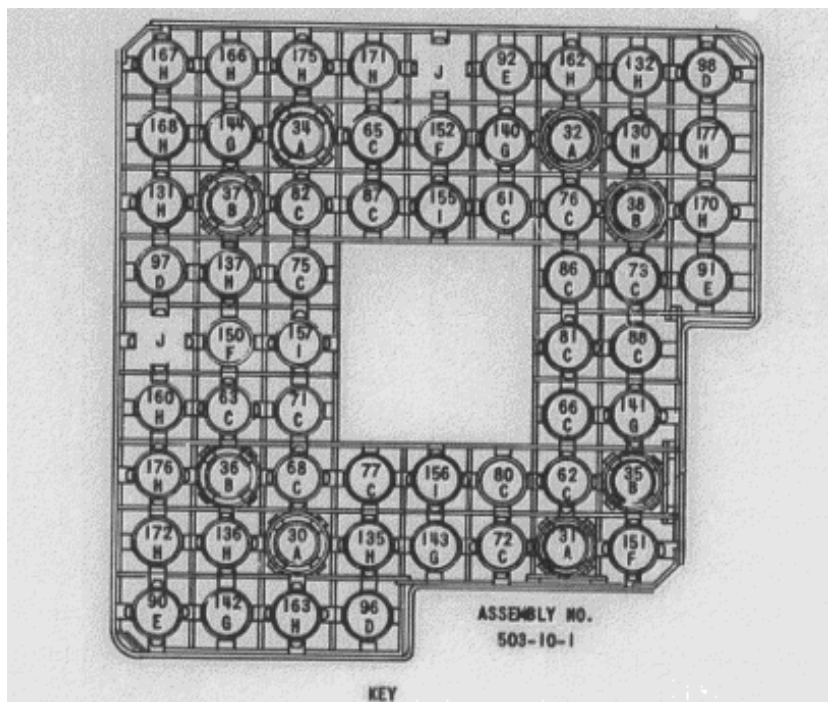


Figure 4: Generic fuel assembly detailed description

2.3. Fuel elements

The fuel elements were assembled into the fuel assemblies by brazing or welding. Thin metal spacers arranged in a grid pattern were placed between fuel elements at appropriate intervals. Fuel-rod details are given in Tables 1A and 1B below, and the plutonium fuel rods are shown in Fig. 5. The alternative, water-tube rod is shown in Fig. 6.

Table 1A Fuel Rod details in the Uranium Assemblies.

Fuel tube outer diameter	0.99314 cm	0.391 in.
Fuel tube inner diameter	0.91694 cm	0.361 in.
Fuel rod pitch	1.4732 cm	0.58 in.
UO₂ enrichment, wt. % (²³⁵U)	5.74	
Clad:	Zircaloy-4 unless noted otherwise in text	
Material		
Thickness	0.0381 cm	0.015 in.
Pellet dimensions:		
Pellet Diameter	0.90678 cm	0.357 in.
Pellet Length	1.85928 cm	0.732 in.
UO₂ density, g/cm³:		
Theoretical	10.96	
Average in the uranium fueled rods	10.18 (~93 % theoretical density))	

Table 1B Fuel Rod details in the Plutonium Assemblies.

UO₂	Natural
PuO₂-UO₂ enrichment, wt. % (PuO₂)	6.60
Theoretical density (TD), g/cm³:	
UO₂	10.96
PuO₂	11.46

Fuel Configuration	Pelletized		VIPAC	
Clad type	SS304	Zircalloy	SS304	Zircalloy
Clad:				
ID, mm	9.1694	8.7503	9.1948	8.7503
(in)	(0.3610)	(0.3445)	(0.362)	(0.3445)
Wall thickness, mm	0.381	0.5664	0.3810	0.5664
(in)	(0.0150)	(0.0233)	(0.015)	(0.0223)
OD,mm	9.9314	9.9314	9.9314	9.9314
(in)	(0.3910)	(0.3910)	(0.391)	(0.3910)
Pellet dimensions:				
Pellet diameter, mm	9.0373	8.7503	9.0373	8.7503
(in)	(0.3558)	(0.3445)	(0.3558)	(0.3445)
Pellet length, mm	9.2964	9.2964	9.2964	9.2694
(in)	(0.3660)	(0.3660)	(0.3660)	(0.3660)
Diametral gap, mm	0.1321	0.1803	0.1321	0.1803
(in)	(0.0052)	(0.0071)	(0.0052)	(0.0071)
Fuel column height, mm	929.64	929.64	929.64	929.64
(in)	(36.600)	(36.600)	(36.600)	(36.600)
End gap, mm	20.244	20.244	21.717	21.717
(in)	(0.7970)	(0.7970)	(0.8550)	(0.8550)
Fuel density, % TD	94		87	

2.4. Control rods and chemical shim

The reactor used six offset cruciform-shaped control rods made of an alloy of silver, indium and cadmium (Fig. 7). The control rods had fueled following sections. Reactor control was also achieved by using boric acid dissolved in the primary coolant as chemical shim. Control rod details are shown below. Little information was available on the control rods in the series of reports. We assumed similar shape and information traditionally used by Westinghouse in this time frame. In particular, it was difficult to locate information on the fuel followers.

Number of moveable control rods	6
Control Rod material	80%Ag, 15%In, 5%Cd
Control Rod Shape	Offset cruciform
Control Rod span	14.2367 cm (5.605 in.)
Control Rod Thickness	0.99695 cm (0.3925 in.)
Estimated control rod worth (average per rod, in %)	3.8
Boron concentration (natural boron), ppm:	
<ul style="list-style-type: none"> To shutdown the reactor down to $K = 0.97$ with no rods inserted (all requirements in ppm): Cold/Hot 	2200/2330
<ul style="list-style-type: none"> To control at power with no rods inserted: Unpoisoned Poisoned (Xe & Sr) 	1835 1495
<ul style="list-style-type: none"> Approximate concentration for one percent worth: Cold/Hot 	80/110

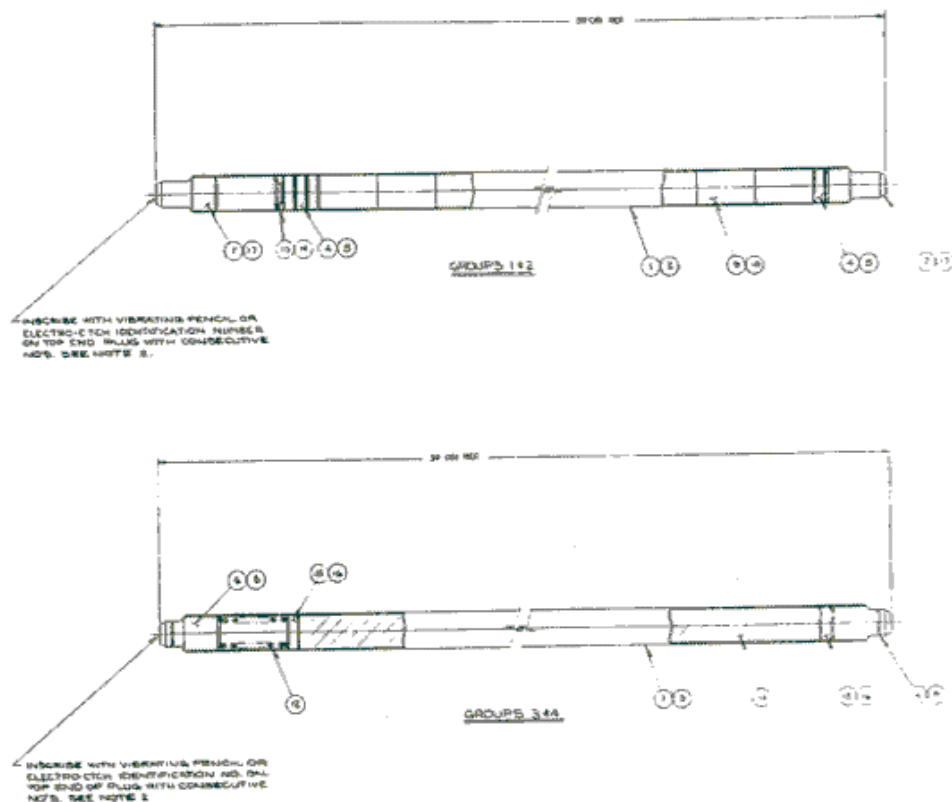


Figure 5: Plutonium rods

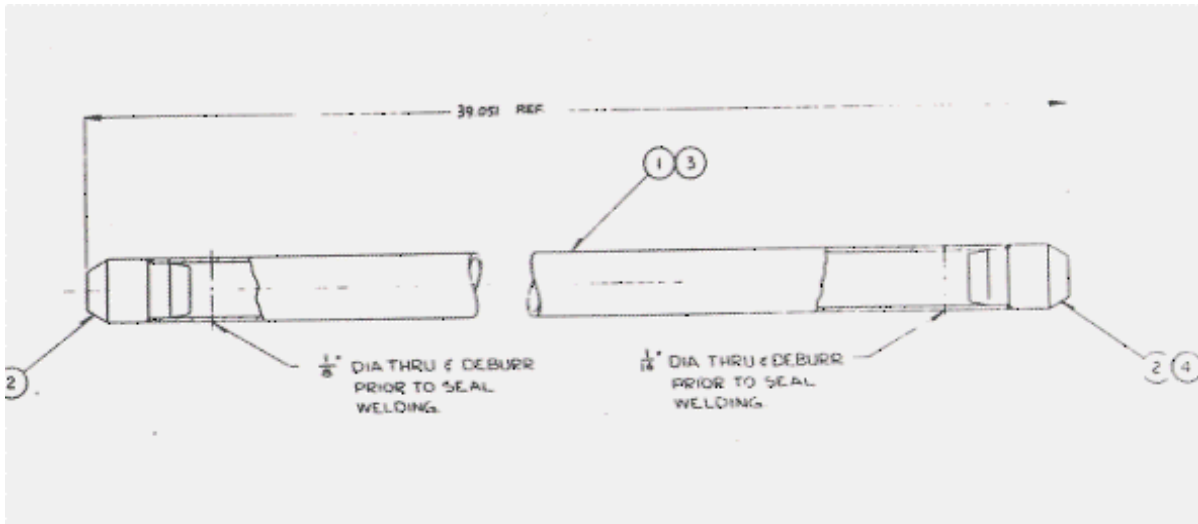


Figure 6: Water tube rod

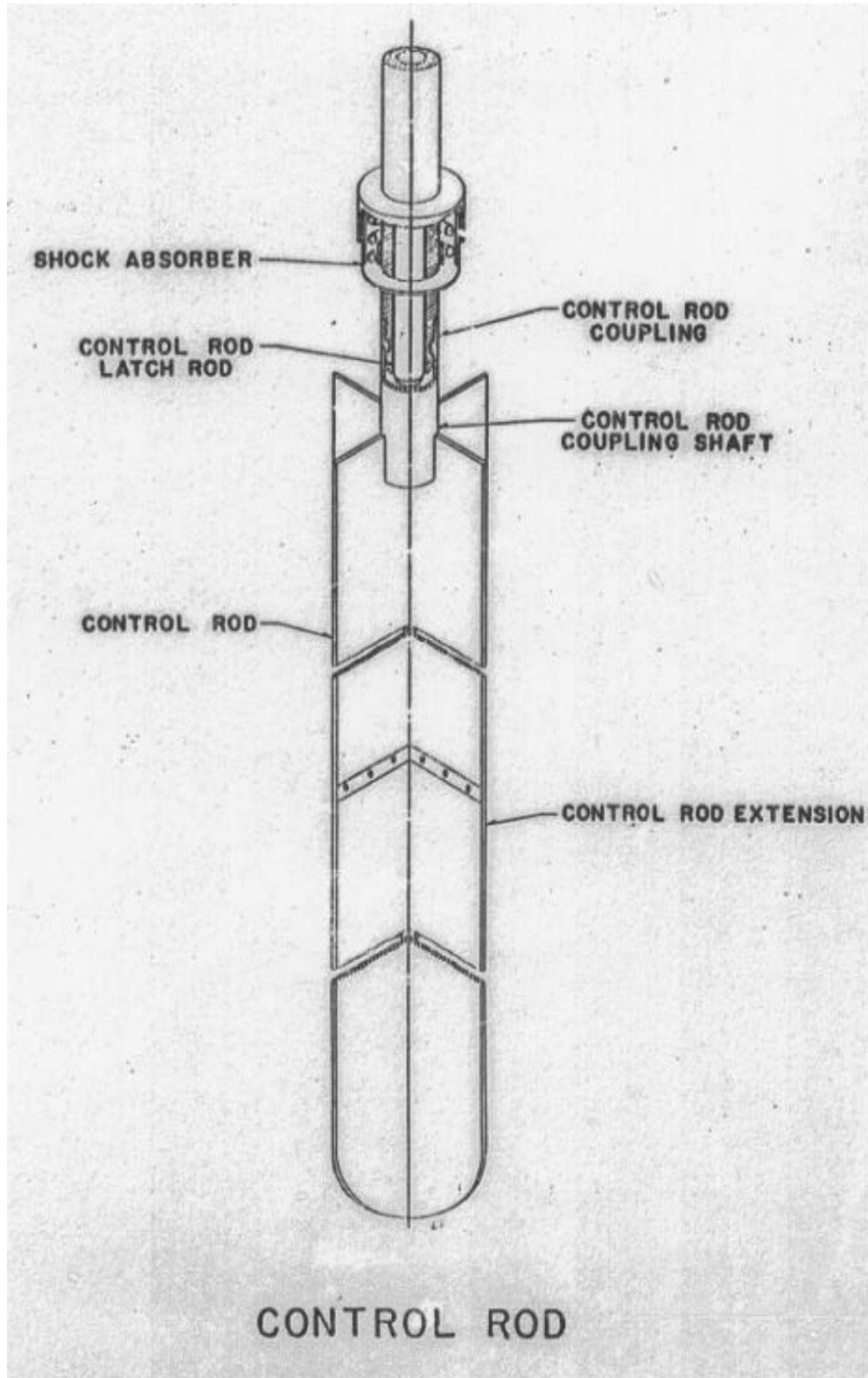


Figure 7: Control rod description. The fueled portion is in the control rod extension.

2.5. Saxton reactor core map

A generic plane view of Saxton Core II is shown in Fig. 8. A similar plan was used for Core I. Fig. 9 shows the generic assembly configuration of Core III.

	B	C	D	E	F
1	B-1 U	C-1 U	D-1 U	E-1 U	F-1 U
2	B-2 U	C-2 P Fuel – PuO ₂ -UO ₂ (Pelletized) Clad – Zircaloy 4	D-2 V Fuel – PuO ₂ -UO ₂ (VIPAC) Clad – Zircaloy 4	E-2 P Fuel – PuO ₂ -UO ₂ (Pelletized) Clad – Zircaloy 4	F-2 U
3	B-3 U	C-3 P Fuel – PuO ₂ -UO ₂ (Pelletized) Clad – Zircaloy 4 (304-SS Clad)	D-3 P Fuel – PuO ₂ -UO ₂ (Pelletized) Clad – Zircaloy 4 (U4, VI – 304-SS Clad, VIPAC M, O – 304-SS Clad, Pelletized)	E-3 P Fuel – PuO ₂ -UO ₂ (Pelletized) Clad – Zircaloy 4	F-3 U
4	B-4 U	C-4 P Fuel – PuO ₂ -UO ₂ (Pelletized) Clad – Zircaloy 4	D-4 V Fuel – PuO ₂ -UO ₂ (VIPAC) Clad – Zircaloy 4 (SS304 Clad)	E-4 P Fuel – PuO ₂ -UO ₂ (Pelletized) Clad – Zircaloy 4	F-4 U
5	B-5 U	C-5 U	D-5 U	E-5 U	F-5 U

Figure 8. Generic floor map of Saxton Core II and assembly specifications. Letters U, P and V represent Uranium, pelletized MOX and VIPAC MOX assemblies respectively.

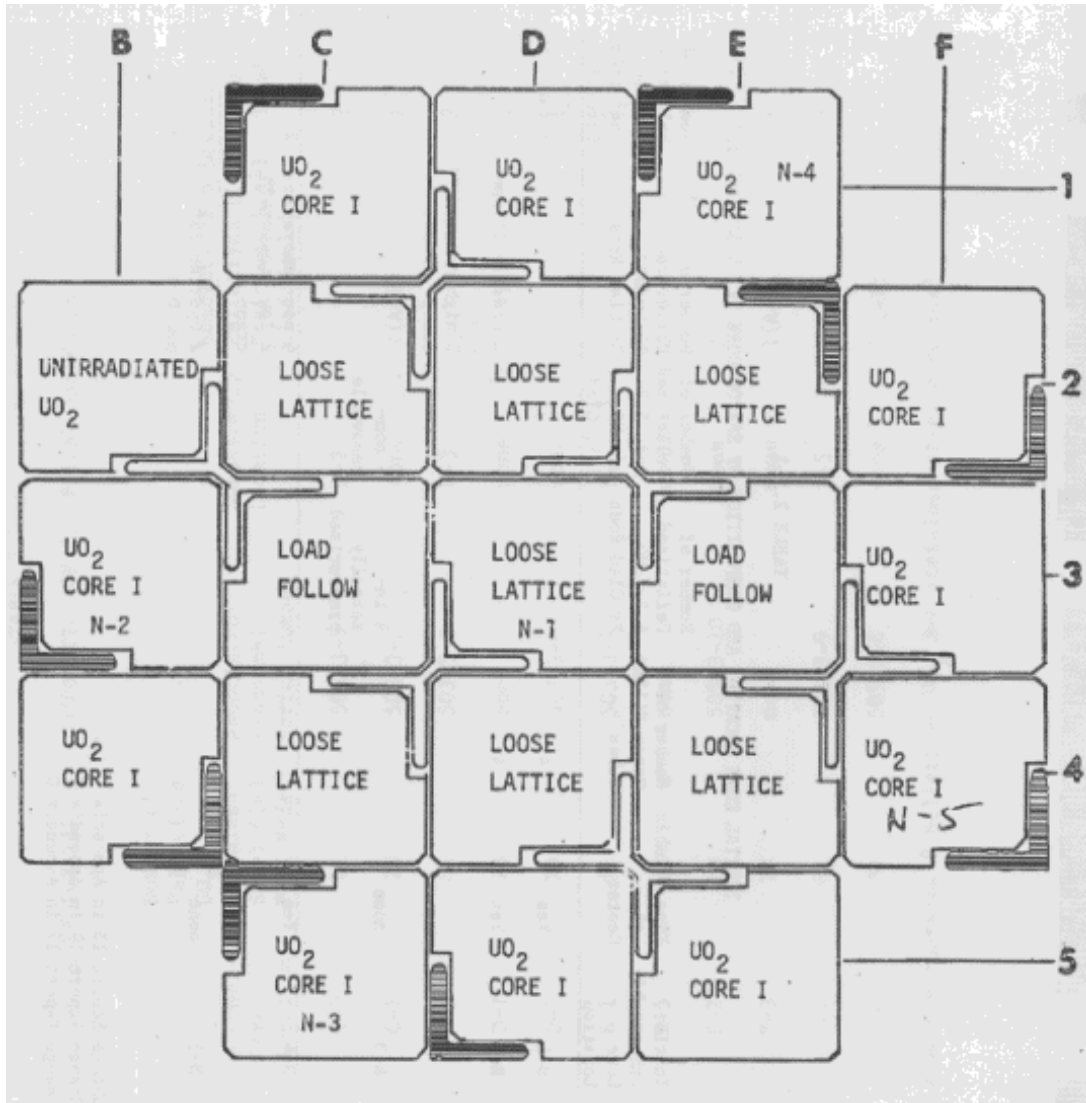


Figure 9. Core III assembly configuration

3. Model description

3.1. Assumptions and simplifications

A generalized model has been created for the Saxton reactor core description. This model describes Saxton Core I, II, and III configurations and enables us to perform consistent analyses of the reactor core characteristics throughout the configurations studied within the frame of the Saxton Plutonium Program. The Saxton Core III configuration is shown in Fig. 9.

Because of the extreme complexity of the Saxton reactor core, several simplifications and assumptions were made in the model. Our main assumptions and simplifications are listed below:

- All UO₂-loaded fuel elements are the same and contain fresh pelletized fuel;
- All cells centered on one fuel element have the same fuel element cell pitch;
- The design of fuel assemblies is assumed to be the same for all Saxton reactor core configurations (Core I – III), only the grid material is changed:
 - MOX fuel assemblies – Inconel grid;
 - UO₂ assemblies – Stainless Steel and Inconel grids;
- All control rods are assumed to have the same characteristics as described in section 3.4;
- A constant thermal power level has been assumed for the fuel lifetime of the Saxton reactor core (detailed power history description can be found in the Saxton project reports but this information was not used within the framework of the present study. The use of this information is discussed further in this report);
- A simplified uniform temperature distribution has been assumed for the Saxton reactor core as described in the sections that follow. (The detailed temperature distribution within the Saxton reactor core can be found in the Saxton project reports but this information was not applied within the framework of the present study);
- All specific characteristics of the Saxton reactor core are implemented into the model as described in sections 3.2 to 3.5.

3.2. Fuel elements

Overall, five different types of fuel element may be found in the Saxton Cores I, II, and III:

- Type 001 (UOX, S): UO₂-loaded fuel elements, fresh fuel;
- Type 002 (MOX, P, Z4): PuO₂-UO₂-loaded fuel elements, pelletized fuel;
- Type 003 (MOX, V, Z4): PuO₂-UO₂-loaded fuel elements, VIPAC fuel;
- Type 004 (MOX, V, S): PuO₂-UO₂-loaded fuel elements, VIPAC fuel;
- Type 005 (MOX, P, S): PuO₂-UO₂-loaded fuel elements, pelletized fuel.

All these types are taken into account in the applied model of the Saxton Core. It is presumed, that Saxton Core I contains only fuel elements of the first type 001, and Saxton Cores II and III contain fuel elements of all five types. Detailed descriptions of all fuel element types are provided in the following pages.

Type 001 (UOX, S): UO₂-loaded fuel elements, fresh fuel

Fuel: UO₂, 10.18 g/cm³, 5.74 wt. % ²³⁵U; fuel region diameter, cm (PD1) - 0.90678

Clad: SS304, 7.94 g/cm³; clad ID, cm (CID1) - 0.91694, clad OD, cm (COD) - 0.99314

Pitch: pitch, cm (p) – 1.47320

Characteristics of the cell centered on one fuel element:

Fuel element cell pitch, cm (p): 1.47320
 Clad OD, cm (COD): 0.99314
 Clad ID, cm (CID1): 0.91694
 Clad wall thickness, cm (CWT1): 0.03810
 Pellet diameter, cm (PD1): 0.90678

UOX fuel characteristics:

UO₂ theoretical density (TD), g/cm³: 10.9600
 % Theoretical density (TD): 92.8832, pelletized
 UO₂ enrichment, wt. % (²³⁵U): 5.74000
 Average UO₂ density, g/cm³: 10.1800

Clad material: SS304
 Clad material density, g/cm³: 7.94000

Clad region composition (type 304-SS , 7.94 g/cm³):

Component	Wt. %	Symbol	ID	Isotopic distribution, wt. %		Atomic mass, amu	Atomic density(b ⁻¹ cm ⁻¹)
Iron	68.375	Fe	26000	26054	5.6990	53.93961480	3.413080E-3
				26056	91.870	55.93494210	5.502012E-2
				26057	2.1410	56.93539870	1.282226E-3
				26058	0.2900	57.93328050	1.736784E-4
Chromium	19.000	Cr	24000	24050	4.1740	49.94604960	6.946348E-4
				24052	83.700	51.94051190	1.392931E-2
				24053	9.6730	52.94065380	1.609775E-3
				24054	2.4530	53.93888490	4.082269E-4
Nickel	9.5000	Ni	28000	28058	67.395	57.93534790	5.607920E-3
				28060	26.653	59.93079060	2.217789E-3
				28061	1.1730	60.93106040	9.760501E-5
				28062	3.7880	61.92834880	3.151984E-4
				28064	0.9910	63.92796960	8.246084E-5
Manganese	2.0000	Mn	25000	25055	100.00	54.93804960	1.751784E-3
Silicon	1.0000	Si	14000	14028	91.873	27.97692650	8.047081E-4
				14029	4.8180	28.97649470	4.220047E-5
				14030	3.3080	29.97377020	2.897450E-5
Carbon	0.0800	C	6000	6012	98.809	12.00000000	6.923680E-5
				6013	1.1910	13.00335480	8.345498E-7
Phosphorus	0.0450	P	15000	15031	100.00	30.97376150	3.941513E-5

Fuel region composition (UO₂ fresh fuel composition):

Component	Symbol	ID	Isotopic distribution, wt. %	Atomic mass, amu	Atomic density (b ⁻¹ cm ⁻¹)
Uranium	²³⁵ U	9223	5.7400	235.0439231	1.303954E-3
	²³⁵ U	5	94.260	238.0507826	2.141301E-2
		9223 8			
Oxygen	O	8016	100.00	15.99491460	4.543393E-2

Type 002 (MOX, P, Z4): PuO₂-UO₂-loaded fuel elements, pelletized fuel

Fuel: 6.6 wt. % PuO₂-UO₂ (natural), pelletized, 9.5692 g/cm³, fuel region diameter, cm (PD2) - 0.87503

Clad: Zircaloy-4, 6.58 g/cm³; clad ID, cm (CID2) - 0.87503, clad OD, cm (COD) - 0.99314

Pitch: pitch, cm (p) - 1.47320

Characteristics of the cell centered on one fuel element:

Fuel element cell pitch, cm (p): 1.473200
 Clad OD, cm (COD): 0.993140
 Clad ID, cm (CID2): 0.875030
 Clad wall thickness, cm (CWT2): 0.059055
 Pellet diameter, cm (PD2): 0.875030

MOX fuel characteristics:

UO₂ theoretical density (TD), g/cm³: 10.96000
 PuO₂ theoretical density (TD), g/cm³: 11.46000
 % Theoretical density (TD): 94, pelletized
 PuO₂-UO₂ enrichment, wt. % (PuO₂): 6.600000
 PuO₂ isotopic distribution, wt. %:
 ²³⁹Pu 90.49000
 ²⁴⁰Pu 8.570000
 ²⁴¹Pu 0.890000
 ²⁴²Pu 0.040000

Average PuO₂-UO₂ density, g/cm³: 10.33342

Clad material: Zircaloy-4

Clad material density, g/cm³: 6.580000

Clad region composition (Zircaloy-4, 6.58 g/cm³):

Component	Wt. %	Symbol	ID	Isotopic distribution, wt. %		Atomic mass, amu	Atomic density (b ⁻¹ cm ⁻¹)
Zirconium	98.200	Zr	40000	40090	50.707	89.90470370	2.19483E-02
				40091	11.181	90.90564500	4.78636E-03
				40092	17.278	91.90504010	7.31593E-03
				40094	17.891	93.90631580	7.41405E-03
				40096	2.9440	95.90827570	1.19453E-03
Tin	1.5000	Sn	50000	50112	0.9140	111.9048208	4.85504E-04
				50114	0.6240	113.9027818	3.25646E-04
				50115	0.3480	114.9033460	1.80029E-04
				50116	14.186	115.9017441	7.27554E-03
				50117	7.5630	116.9029538	3.84560E-03
				50118	24.055	117.9016063	1.21278E-02
				50119	8.5940	118.9033089	4.29632E-03
				50120	32.917	119.9021966	1.63188E-02
				50122	4.7550	121.9034401	2.31862E-03
50124	6.0430	123.9052746	2.89907E-03				
Iron	0.2000	Fe	26000	26054	5.6990	53.93961480	8.37385E-04
				26056	91.870	55.93494210	1.30174E-02
				26057	2.1410	56.93539870	2.98036E-04
				26058	0.2900	57.93328050	3.96738E-05
Chromium	0.1000	Cr	24000	24050	4.1740	49.94604960	3.31173E-04
				24052	83.700	51.94051190	6.38592E-03
				24053	9.6730	52.94065380	7.24062E-04
				24054	2.4530	53.93888490	1.80219E-04

Fuel region composition (PuO₂-UO₂ fuel composition, pelletized):

Component	Symbol	ID	Isotopic distribution, wt. %	Atomic mass, amu	Atomic density (b ⁻¹ cm ⁻¹)
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Plutonium	²³⁹ Pu	9423	90.490	239.0521565	1.376016E-03
	²⁴⁰ Pu	9	8.5700	240.0538075	1.303178E-04
	²⁴¹ Pu	9424	0.8900	241.0568453	1.353359E-05
	²⁴² Pu	0	0.0400	242.0587368	6.082511E-07
		9424 1 9424 2			
Uranium	²³⁵ U	9223	0.7160	235.0439231	1.540774E-04
	²³⁵ U	5	99.284	238.0507826	2.136511E-02
		9223			
		8			
Oxygen	O	8016	100.00	15.99491460	4.607963E-02

Type 003 (MOX, V, Z4): PuO₂-UO₂-loaded fuel elements, VIPAC fuel

Fuel: 6.6 wt. % PuO₂-UO₂ (natural), VIPAC, 8.8566 g/cm³, fuel region diameter, cm (PD2) - 0.87503

Clad: Zircaloy-4, 6.58 g/cm³; clad ID, cm (CID2) - 0.87503, clad OD, cm (COD) - 0.99314

Pitch: pitch, cm (p) - 1.47320

Characteristics of the cell centered on one fuel element:

Fuel element cell pitch, cm (p): 1.473200
 Clad OD, cm (COD): 0.993140
 Clad ID, cm (CID2): 0.875030
 Clad wall thickness, cm (CWT3): 0.056642
 Pellet diameter, cm (PD2): 0.875030

MOX fuel characteristics:

UO₂ theoretical density (TD), g/cm³: 10.96000
 PuO₂ theoretical density (TD), g/cm³: 11.46000
 % Theoretical density (TD): 87, VIPAC
 PuO₂-UO₂ enrichment, wt. % (PuO₂): 6.600000
 PuO₂ isotopic distribution, wt. %:
 ²³⁹Pu 90.49000
 ²⁴⁰Pu 8.570000
 ²⁴¹Pu 0.890000
 ²⁴²Pu 0.040000

Average PuO₂-UO₂ density, g/cm³: 9.563910

Clad material: Zircaloy-4
 Clad material density, g/cm³: 6.580000

Clad region composition (Zircaloy-4, 6.58 g/cm³):

Component	Wt. %	Symbol	ID	Isotopic distribution, wt. %		Atomic mass, amu	Atomic density (b ⁻¹ cm ⁻¹)
Zirconium	98.200	Zr	40000	40090	50.707	89.90470370	2.19483E-02
				40091	11.181	90.90564500	4.78636E-03
				40092	17.278	91.90504010	7.31593E-03
				40094	17.891	93.90631580	7.41405E-03
				40096	2.9440	95.90827570	1.19453E-03
Tin	1.5000	Sn	50000	50112	0.9140	111.9048208	4.85504E-04
				50114	0.6240	113.9027818	3.25646E-04
				50115	0.3480	114.9033460	1.80029E-04
				50116	14.186	115.9017441	7.27554E-03
				50117	7.5630	116.9029538	3.84560E-03
				50118	24.055	117.9016063	1.21278E-02
				50119	8.5940	118.9033089	4.29632E-03
				50120	32.917	119.9021966	1.63188E-02
				50122	4.7550	121.9034401	2.31862E-03
50124	6.0430	123.9052746	2.89907E-03				
Iron	0.2000	Fe	26000	26054	5.6990	53.93961480	8.37385E-04
				26056	91.870	55.93494210	1.30174E-02
				26057	2.1410	56.93539870	2.98036E-04
				26058	0.2900	57.93328050	3.96738E-05
Chromium	0.1000	Cr	24000	24050	4.1740	49.94604960	3.31173E-04
				24052	83.700	51.94051190	6.38592E-03
				24053	9.6730	52.94065380	7.24062E-04
				24054	2.4530	53.93888490	1.80219E-04

Fuel region composition (PuO₂-UO₂ fuel composition, VIPAC):

Component	Symbol	ID	Isotopic distribution, wt. %	Atomic mass, amu	Atomic density (b ⁻¹ cm ⁻¹)
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Plutonium	²³⁹ Pu	9423	90.490	239.0521565	1.273547E-03
	²⁴⁰ Pu	9	8.5700	240.0538075	1.206133E-04
	²⁴¹ Pu	9424	0.8900	241.0568453	1.252577E-05
	²⁴² Pu	0	0.0400	242.0587368	5.629558E-07
		9424 1 9424 2			
Uranium	²³⁵ U	9223	0.7160	235.0439231	1.426035E-04
	²³⁵ U	5	99.284	238.0507826	1.977409E-02
		9223 8			
Oxygen	O	8016	100.00	15.99491460	4.264817E-02

Type 004 (MOX, V, S): PuO₂-UO₂-loaded fuel elements, VIPAC fuel

Fuel: 6.6 wt. % PuO₂-UO₂ (natural), VIPAC, 8.8566 g/cm³, fuel region diameter, cm (PD3) - 0.903732

Clad: SS304, 7.94 g/cm³; clad ID, cm (CID3) - 0.91948, clad OD, cm (COD) - 0.99314

Pitch: pitch, cm (p) – 1.47320

Characteristics of the cell centered on one fuel element:

Fuel element cell pitch, cm (p):	1.473200
Clad OD, cm (COD):	0.993140
Clad ID, cm (CID3):	0.919480
Clad wall thickness, cm (CWT1):	0.038100
Pellet diameter, cm (PD3):	0.903732

MOX fuel characteristics:

UO ₂ theoretical density (TD), g/cm ³ :	10.96000	
PuO ₂ theoretical density (TD), g/cm ³ :	11.46000	
% Theoretical density (TD):	87, VIPAC	
PuO ₂ -UO ₂ enrichment, wt. % (PuO ₂):	6.600000	
PuO ₂ isotopic distribution, wt. %:	²³⁹ Pu	90.49000
	²⁴⁰ Pu	8.570000
	²⁴¹ Pu	0.890000
	²⁴² Pu	0.040000
Average PuO ₂ -UO ₂ density, g/cm ³ :	9.563910	

Clad material: SS304

Clad material density, g/cm³: 7.94000

Clad region composition (SS304 (Stainless Steel – 304, 7.94 g/cm³) Composition):

Component	Wt. %	Symbol	ID	Isotopic distribution, wt. %		Atomic mass, amu	Atomic density (b ⁻¹ cm ⁻¹)
Iron	68.375	Fe	2600 0	26054	5.6990	53.93961480	3.413080E-3
				26056	91.870	55.93494210	5.502012E-2
				26057	2.1410	56.93539870	1.282226E-3
				26058	0.2900	57.93328050	1.736784E-4
Chromium	19.000	Cr	2400 0	24050	4.1740	49.94604960	6.946348E-4
				24052	83.700	51.94051190	1.392931E-2
				24053	9.6730	52.94065380	1.609775E-3
				24054	2.4530	53.93888490	4.082269E-4
Nickel	9.5000	Ni	2800 0	28058	67.395	57.93534790	5.607920E-3
				28060	26.653	59.93079060	2.217789E-3
				28061	1.1730	60.93106040	9.760501E-5
				28062	3.7880	61.92834880	3.151984E-4
				28064	0.9910	63.92796960	8.246084E-5
Manganese	2.0000	Mn	2500 0	25055	100.00	54.93804960	1.751784E-3
Silicon	1.0000	Si	1400 0	14028	91.873	27.97692650	8.047081E-4
				14029	4.8180	28.97649470	4.220047E-5
				14030	3.3080	29.97377020	2.897450E-5
Carbon	0.0800	C	6000	6012	98.809	12.00000000	6.923680E-5
				6013	1.1910	13.00335480	8.345498E-7
Phosphorus	0.0450	P	1500 0	15031	100.00	30.97376150	3.941513E-5

Fuel region composition (PuO₂-UO₂ fuel composition, VIPAC):

Component	Symbol	ID	Isotopic distribution, wt. %	Atomic mass, amu	Atomic density (b ⁻¹ cm ⁻¹)
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Plutonium	²³⁹ Pu	9423	90.490	239.0521565	1.273547E-03
	²⁴⁰ Pu	9	8.5700	240.0538075	1.206133E-04
	²⁴¹ Pu	9424	0.8900	241.0568453	1.252577E-05
	²⁴² Pu	0	0.0400	242.0587368	5.629558E-07
		9424			
		1			
		9424			
		2			
Uranium	²³⁵ U	9223	0.7160	235.0439231	1.426035E-04
	²³⁵ U	5	99.284	238.0507826	1.977409E-02
		9223			
		8			
Oxygen	O	8016	100.00	15.99491460	4.264817E-02

Type 005 (MOX, P, S): PuO₂-UO₂-loaded fuel elements, pelletized fuel

Fuel: 6.6 wt. % PuO₂-UO₂ (natural), pelletized, 9.5692 g/cm³, fuel region diameter, cm (PD3) - 0.903732

Clad: SS304, 7.94 g/cm³; clad ID, cm (CID1) - 0.91694, clad OD, cm (COD) - 0.99314

Pitch: pitch, cm (p) - 1.47320

Characteristics of the cell centered on one fuel element:

Fuel element cell pitch, cm (p): 1.473200
 Clad OD, cm (COD): 0.993140
 Clad ID, cm (CID1): 0.916940
 Clad wall thickness, cm (CWT1): 0.038100
 Pellet diameter, cm (PD3): 0.903732

MOX fuel characteristics:

UO₂ theoretical density (TD), g/cm³: 10.96000
 PuO₂ theoretical density (TD), g/cm³: 11.46000
 % Theoretical density (TD): 94, pelletized
 PuO₂-UO₂ enrichment, wt. % (PuO₂): 6.600000
 PuO₂ isotopic distribution, wt. %:
 ²³⁹Pu 90.49000
 ²⁴⁰Pu 8.570000
 ²⁴¹Pu 0.890000
 ²⁴²Pu 0.040000

Average PuO₂-UO₂ density, g/cm³: 10.33342

Clad material: 304-SS

Clad material density, g/cm³: 7.94000

Clad region composition (304-SS- 304, 7.94 g/cm³):

Component	Wt. %	Symbol	ID	Isotopic distribution, wt. %		Atomic mass, amu	Atomic density (b ⁻¹ cm ⁻¹)
Iron	68.375	Fe	2600 0	26054	5.6990	53.93961480	3.413080E-3
				26056	91.870	55.93494210	5.502012E-2
				26057	2.1410	56.93539870	1.282226E-3
				26058	0.2900	57.93328050	1.736784E-4
Chromium	19.000	Cr	2400 0	24050	4.1740	49.94604960	6.946348E-4
				24052	83.700	51.94051190	1.392931E-2
				24053	9.6730	52.94065380	1.609775E-3
				24054	2.4530	53.93888490	4.082269E-4
Nickel	9.5000	Ni	2800 0	28058	67.395	57.93534790	5.607920E-3
				28060	26.653	59.93079060	2.217789E-3
				28061	1.1730	60.93106040	9.760501E-5
				28062	3.7880	61.92834880	3.151984E-4
				28064	0.9910	63.92796960	8.246084E-5
Manganese	2.0000	Mn	2500 0	25055	100.00	54.93804960	1.751784E-3
Silicon	1.0000	Si	1400 0	14028	91.873	27.97692650	8.047081E-4
				14029	4.8180	28.97649470	4.220047E-5
				14030	3.3080	29.97377020	2.897450E-5
Carbon	0.0800	C	6000	6012	98.809	12.00000000	6.923680E-5
				6013	1.1910	13.00335480	8.345498E-7
Phosphorus	0.0450	P	1500 0	15031	100.00	30.97376150	3.941513E-5

Fuel region composition (PuO₂-UO₂ fuel composition, pelletized):

Component	Symbol	ID	Isotopic distribution, wt. %	Atomic mass, amu	Atomic density (b ⁻¹ cm ⁻¹)
-----------	--------	----	------------------------------	------------------	--

Plutonium	²³⁹ Pu	9423	90.490	239.0521565	1.376016E-03
	²⁴⁰ Pu	9	8.5700	240.0538075	1.303178E-04
	²⁴¹ Pu	9424	0.8900	241.0568453	1.353359E-05
	²⁴² Pu	0	0.0400	242.0587368	6.082511E-07
		9424			
		1			
	9424				
	2				
Uranium	²³⁵ U	9223	0.7160	235.0439231	1.540774E-04
	²³⁵ U	5	99.284	238.0507826	2.136511E-02
		9223			
		8			
Oxygen	O	8016	100.00	15.99491460	4.607963E-02

Technical summary from the above complete description of the fuel elements is given in Table 2 and shown in Figure 9. Special notation was established in order to define characteristics of the fuel elements and distinguish between different types of the fuel elements.

Table 2 Types of Fuel Elements

a) Notation

Fuel element	UOX, S	MOX, P, Z4	MOX, V, Z4	MOX, P, S	MOX, V, S
Fuel element cell pitch, in	P=0.58				
Fuel region diameter, in	PD1=0.357	PD2=0.3445		PD3=0.3558	
Clad:					
Outside diameter, in	COD=0.391	COD=0.391	COD=0.391	COD=0.391	COD=0.391
Inside diameter, in	CID1=0.361	CID2=0.3445	CID2=0.3445	CID1=0.361	CID3=0.362
Thickness, in	CWT1=0.015	CWT2=0.0233	CWT3=0.0223	CWT1=0.015	CWT1=0.015

b) Summarized characteristics

Type of Fuel Element	FE001	FE002	FE003	FE005	FE004
Description	UOX, S	MOX, P, Z4	MOX, V, Z4	MOX, P, S	MOX, V, S
Fuel element cell pitch, cm	p=1.47320				
Fuel region:					
Diameter, cm	PD1=0.9067800	PD2=0.87503	PD2=0.87503	PD3=0.903732	PD3=0.903732
Fuel,	UO ₂ ,	PuO ₂ - UO ₂ ,	PuO ₂ - UO ₂ ,	PuO ₂ - UO ₂ ,	PuO ₂ - UO ₂ ,
Enrichment, wt %	5.74 (²³⁵ U)	6.6 (PuO ₂)	6.6 (PuO ₂)	6.6 (PuO ₂)	6.6 (PuO ₂)
Density, g/cm ³	10.18	10.33342	9.563910	10.33342	9.563910
Composition (Nuclei/(b*cm)):					
94239	-	1.376016E-03	1.273547E-03	1.376016E-03	1.273547E-03
94240	-	1.303178E-04	1.206133E-04	1.303178E-04	1.206133E-04
94241	-	1.353359E-05	1.252577E-05	1.353359E-05	1.252577E-05
94242	-	6.082511E-07	5.629558E-07	6.082511E-07	5.629558E-07
92235	1.303954E-3	1.540774E-04	1.426035E-04	1.540774E-04	1.426035E-04
92238	2.141301E-2	2.136511E-02	1.977409E-02	2.136511E-02	1.977409E-02
8016	4.543393E-2	4.607963E-02	4.264817E-02	4.607963E-02	4.264817E-02
Clad:					
OD, cm	COD=0.993140	COD=0.993140	COD=0.993140	COD=0.993140	COD=0.993140
ID, cm	CID1=0.9169400	CID2=0.8750300	CID2=0.8750300	CID1=0.9169400	CID3=0.9194800
Thickness, cm	CWT1=0.038100	CWT2=0.059055	CWT3=0.056642	CWT1=0.038100	CWT1=0.038100
Material	SS304,	Zircaloy-4,	Zircaloy-4,	SS304,	SS304,
Density, g/cm ³	7.94	6.58	6.58	7.94	7.94
Composition (Nuclei/(b*cm)):					
Zirconium					
40090	-	2.159841E-02	2.159841E-02	-	-
40091	-	4.762495E-03	4.762495E-03	-	-
40092	-	7.359484E-03	7.359484E-03	-	-
40094	-	7.620589E-03	7.620589E-03	-	-
40096	-	1.253983E-03	1.253983E-03	-	-
Tin					
50112	-	5.946753E-06	5.946753E-06	-	-
50114	-	4.059928E-06	4.059928E-06	-	-
50115	-	2.264190E-06	2.264190E-06	-	-
50116	-	9.229829E-05	9.229829E-05	-	-
50117	-	4.920710E-05	4.920710E-05	-	-
50118	-	1.565089E-04	1.565089E-04	-	-
50119	-	5.591509E-05	5.591509E-05	-	-
50120	-	2.141677E-04	2.141677E-04	-	-
50122	-	3.093743E-05	3.093743E-05	-	-
50124	-	3.931754E-05	3.931754E-05	-	-

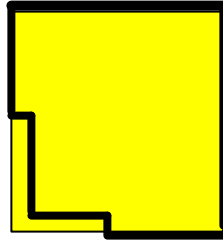
Iron	26054	3.413080E-3	4.943916E-06	4.943916E-06	3.413080E-3	3.413080E-3
	26056	5.502012E-2	7.969777E-05	7.969777E-05	5.502012E-2	5.502012E-2
	26057	1.282226E-3	1.857330E-06	1.857330E-06	1.282226E-3	1.282226E-3
	26058	1.736784E-4	2.515767E-07	2.515767E-07	1.736784E-4	1.736784E-4
Chromium	24050	6.946348E-4	1.810485E-06	1.810485E-06	6.946348E-4	6.946348E-4
	24052	1.392931E-2	3.630512E-05	3.630512E-05	1.392931E-2	1.392931E-2
	24053	1.609775E-3	4.195692E-06	4.195692E-06	1.609775E-3	1.609775E-3
	24054	4.082269E-4	1.063996E-06	1.063996E-06	4.082269E-4	4.082269E-4
Nickel	28058	5.607920E-3	-	-	5.607920E-3	5.607920E-3
	28060	2.217789E-3	-	-	2.217789E-3	2.217789E-3
	28061	9.760501E-5	-	-	9.760501E-5	9.760501E-5
	28062	3.151984E-4	-	-	3.151984E-4	3.151984E-4
	28064	8.246084E-5	-	-	8.246084E-5	8.246084E-5
Manganese	25055	1.751784E-3	-	-	1.751784E-3	1.751784E-3
Silicon	14028	8.047081E-4	-	-	8.047081E-4	8.047081E-4
	14029	4.220047E-5	-	-	4.220047E-5	4.220047E-5
	14030	2.897450E-5	-	-	2.897450E-5	2.897450E-5
Carbon	6012	6.923680E-5	-	-	6.923680E-5	6.923680E-5
	6013	8.345498E-7	-	-	8.345498E-7	8.345498E-7
Phosphorus	15031	3.941513E-5	-	-	3.941513E-5	3.941513E-5

P – Pelletized fuel; V – VIPAC fuel; S – SS304 clad; Z4 – Zircaloy-4

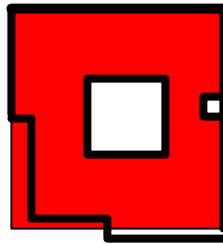
3.3. Fuel assemblies

Eight different types of fuel assembly geometry have been found in the Saxton Core I and Saxton Core II:

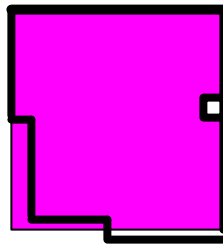
- **Type 100:**



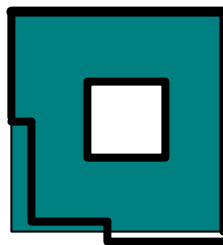
- **Type 200:**



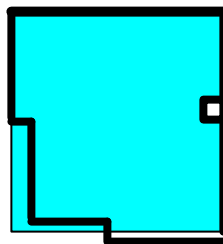
- **Type 300:**



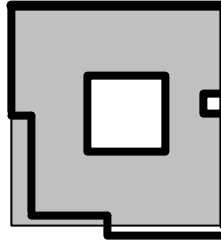
- **Type 400:**



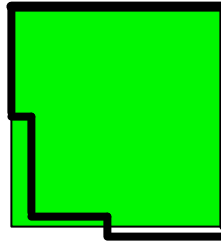
- **Type 500:**



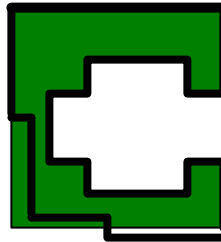
- Type 600:



- Type 700:



- Type 800:



Locations of the fuel assemblies of each type are shown in Figure 10 below. All these types of geometry are taken into account explicitly in the Saxton Core Model.

	B	C	D	E	F
1	B1	C1 Type: 700 Fuel element: 001	D1 Type: 300 Fuel element: 001	E1 Type: 800 Fuel element: 001	F1
2	B2 Type: 100 Fuel element: 001	C2 Type: 100 Fuel element: 002	D2 Type: 100 Fuel element: 003	E2 Type: 700 Fuel element: 002	F2 Type: 100 Fuel element: 001
3	B3 Type: 200 Fuel element: 001	C3 Type: 500 Fuel element: 002, 005	D3 Type: 200 Fuel element: 002, 004, 005	E3 Type: 300 Fuel element: 002	F3 Type: 700 Fuel element: 001
4	B4 Type: 300 Fuel element: 001	C4 Type: 100 Fuel element: 002	D4 Type: 500 Fuel element: 003, 004	E4 Type: 100 Fuel element: 002	F4 Type: 600 Fuel element: 001
5	B5	C5 Type: 400 Fuel element: 001	D5 Type: 300 Fuel element: 001	E5 Type: 100 Fuel element: 001	F5

Figure 10. Types of the fuel assembly geometry in the Model of Saxton Core II

Out of the 12 UO_2 assemblies, five were original Core I design (SS grids) and seven were new, Core II design (Inconel grids). All the mixed oxide ($\text{PuO}_2\text{-UO}_2$) assemblies contained inconel grids.

Within the framework of the present study the following materials were used for the assembly:

- Stainless Steel (304-SS)
- INCONEL
- Aluminum

Material compositions are listed below:

Stainless Steel grid:

Type 304-SS, 7.94 g/cm³:

Component	Wt. %	Symbol	ID	Isotopic distribution, wt. %		Atomic mass, amu	Atomic density (b ⁻¹ cm ⁻¹)
Iron	68.375	Fe	26000	2605	5.6990	53.93961480	3.45429E-03
				4	91.870	55.93494210	5.36981E-02
				2605	2.1410	56.93539870	1.22943E-03
				6	0.2900	57.93328050	1.63659E-04
				2605			
				7			
				2605			
				8			
Chromium	19.000	Cr	24000	2405	4.1740	49.94604960	7.59235E-04
				0	83.700	51.94051190	1.46401E-02
				2405	9.6730	52.94065380	1.65996E-03
				2	2.4530	53.93888490	4.13162E-04
				2405			
				3			
Nickel	9.5000	Ni	28000	2805	67.395	57.93534790	5.28420E-03
				8	26.653	59.93079060	2.02018E-03
				2806	1.1730	60.93106040	8.74489E-05
				0	3.7880	61.92834880	2.77853E-04
				2806	0.9910	63.92796960	7.04171E-05
				1			
				2806			
				2			
Manganese	2.0000	Mn	25000	2505	100.00	54.93804960	1.74072E-03
				5			
Silicon	1.0000	Si	14000	1402	91.873	27.97692650	1.57021E-03
				8	4.8180	28.97649470	7.95046E-05
				1402	3.3080	29.97377020	5.27710E-05
				9			
				1403			
Carbon	0.0800	C	6000	6012	98.809	12.00000000	3.14975E-04
				6013	1.1910	13.00335480	3.50363E-06
Phosphorus	0.0450	P	15000	1503	100.00	30.97376150	6.94688E-05
			0	1			

Inconel grid:

INCONEL (8.30 g/cm³) Composition:

Component	Wt. %	Symbol	ID	Isotopic distribution, wt. %	Atomic mass, amu	Atomic density (b ⁻¹ cm ⁻¹)
-----------	-------	--------	----	------------------------------	------------------	--

Iron	7.0000	Fe	2600 0	2605 4 2605 6 2605 7 2605 8	5.6990 91.870 2.1410 0.2900	53.93961480 55.93494210 56.93539870 57.93328050	3.69673E-04 5.74669E-03 1.31571E-04 1.75145E-05
Chromium	15.000	Cr	2400 0	2405 0 2405 2 2405 3 2405 4	4.1740 83.700 9.6730 2.4530	49.94604960 51.94051190 52.94065380 53.93888490	6.26573E-04 1.20820E-02 1.36991E-03 3.40970E-04
Nickel	73.000	Ni	2800 0	2805 8 2806 0 2806 1 2806 2 2806 4	67.395 26.653 1.1730 3.7880 0.9910	57.93534790 59.93079060 60.93106040 61.92834880 63.92796960	4.24459E-02 1.62274E-02 7.02443E-04 2.23189E-03 5.65633E-04
Silicon	2.5000	Si	1400 0	1402 8 1402 9 1403 0	91.873 4.8180 3.3080	27.97692650 28.97649470 29.97377020	4.10352E-03 2.07773E-04 1.37909E-04
Titanium	2.5000	Ti	2200 0	2204 6 2204 7 2204 8 2204 9 2205 0	7.6780 7.1580 73.903 5.6220 5.6390	45.95262950 46.95176380 47.94794710 48.94787080 49.94479210	2.08789E-04 1.90506E-04 1.92602E-03 1.43524E-04 1.41085E-04

Aluminum:Aluminum (2.702 g/cm³) Composition:

Component	Wt. %	Symbol	ID	Isotopic distribution, wt. %		Atomic mass, amu	Atomic density (b ⁻¹ cm ⁻¹)
Iron	100.00	Al	13000	13027	100.00	26.98180000	6.03067E-2

3.4. Control rods

Description:

The reactor used six offset cruciform shaped control rods made of an alloy of silver, indium and cadmium. The control rods had fueled follow-on sections. Details of the control rods are given below.

Number: 6**Type:** Offset cruciform shaped control rod**Material:** An alloy of silver, indium, and cadmium

Characteristics of the control rod:

Shape: Offset cruciform

Span, cm (CRS): 14.23670

Half-span, cm (half-CRS): 7.118350

Thickness, cm (CRT): 0.996950

Half-thickness, cm (half-CRT): 0.498475

Alloy components distribution, wt. %:

Ag (silver) 80.00000, theoretical density – 10.50 g/cm³In (indium) 15.00000, theoretical density – 7.300 g/cm³Cd (cadmium) 5.000000, theoretical density – 8.642 g/cm³Control rod average material density, g/cm³: 9.927100

Control rod composition:

Component	Wt. %	Symbol	ID	Isotopic distribution, wt. %		Atomic mass, amu	Atomic density (b ⁻¹ cm ⁻¹)

Silver	80.000	Ag	4700 0	4710 7 4710 9	51.377 48.623	106.9050930 108.9047555	2.29844E-02 2.13530E-02
Indium	15.000	In	4900 0	4911 3 4911 5	4.2280 95.772	112.9040612 114.9038783	3.35807E-04 7.47426E-03
Cadmium	5.0000	Cd	4800 0	4810 6 4810 8 4811 0 4811 1 4811 2 4811 3 4811 4 4811 6	1.1780 0.8540 12.211 12.628 24.021 12.273 29.111 7.7230	105.9064580 107.9041834 109.9030056 110.9041816 111.9027572 112.9044009 113.9033581 115.9047554	3.32480E-05 2.36572E-05 3.32112E-04 3.40353E-04 6.41643E-04 3.24925E-04 7.63948E-04 1.99172E-04

3.5. Coolant

Type of coolant: H₂O

Characteristics of the coolant:

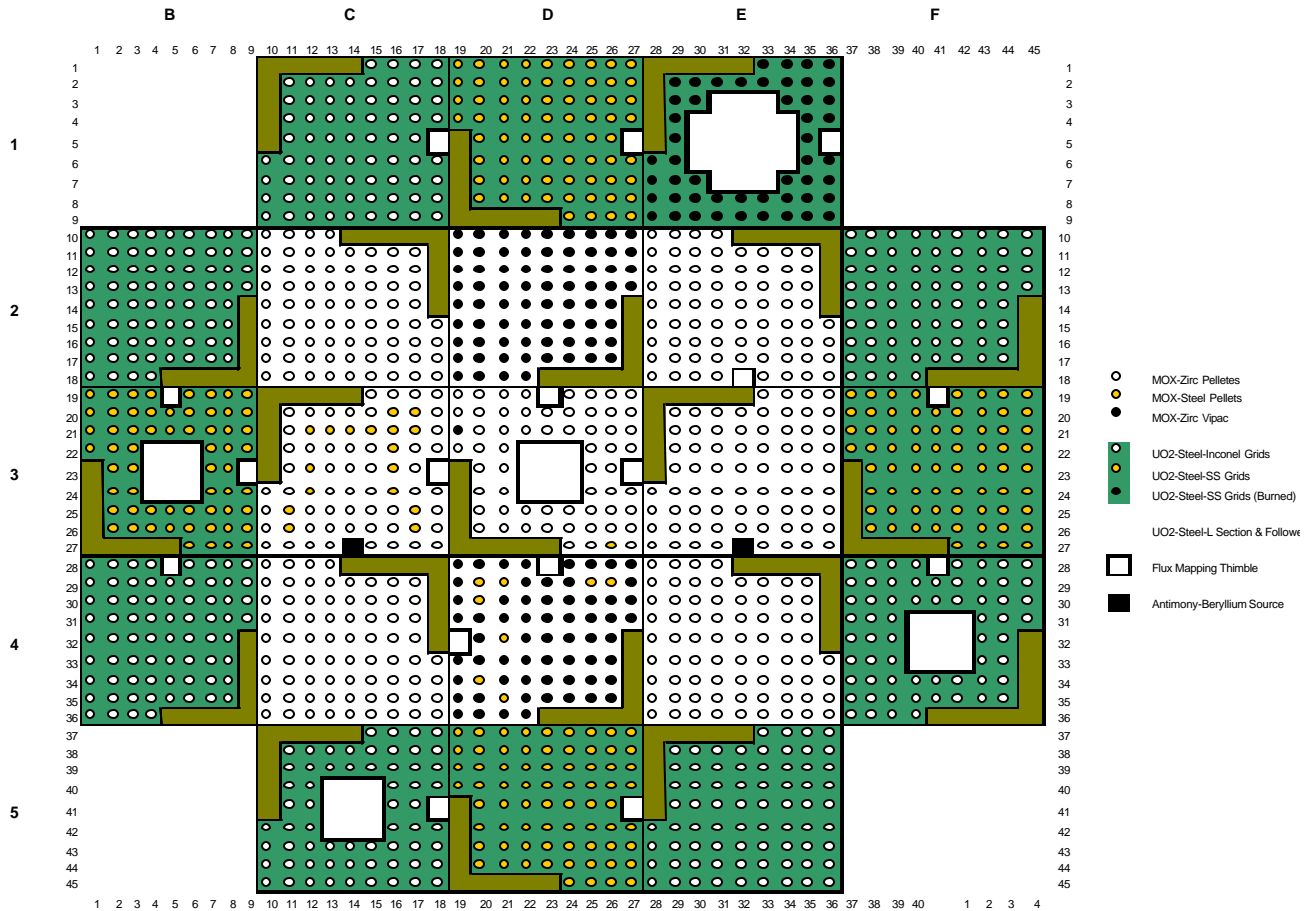
System pressure (nominal), MPa:	13.789514586
Average coolant temperature, K:	551.483
Coolant average density, g/cm ³ :	0.7656

Coolant composition:

Component	Symbol	ID	Natural Element, amu	Isotopic distribution, wt. %	Atomic mass, amu	Atomic density ($b^{-1}cm^{-1}$)
Hydrogen	H	1001	1.00794	100.000	1.007825000	5.11983E-02
Oxygen	O	8016	15.9994	100.000	15.99491460	2.559914E-02

3.6. Reactor core

The layout of Saxton Core II is given below:



Characteristics of the SAXTON partial plutonium core:

Thermal power, MWt:	23.5
Fuel lifetime, hours	7300 - 8400
Temperature, K:	
Fuel temperature:	810.928
Clad temperature:	Zircaloy – 591.483/SS304 – 588.706
Average coolant temperature:	551.483

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4. HELIOS Input data specification

4.1. General sequence of calculations with the HELIOS computer codes system

The HELIOS computer code system (version 1.4) provides the capability to perform neutron and gamma transport calculations for lattice burn-up in general two-dimensional geometry. This code system consists of four functional components:

- AURORA – input processor;
- HELIOS – general 2D geometry coupled neutron-gamma transport code for lattice burn-up;
- ZENITH – output processor;
- HERMES – database and subroutine package.

The HELIOS system execution starts with an AURORA run. AURORA prepares technical HELIOS input information on the basis of the user's input file. This technical information is then stored in the HERMES database in the form applicable for utilization by HELIOS and ZENITH. HELIOS then takes the input data from HERMES and performs the set of requested calculations. All results of HELIOS calculations are also stored in the HERMES database for further processing by ZENITH, which converts output information to the form requested by user.

4.2. Nuclear data library

The nuclear data library for HELIOS has the following characteristics (note that NO stands for "Number Of" in the following listing):

• TECHNICAL IDENTIFICATION:		"HEBE", Version: 1.4, July 29 1996
• LIBRARY IDENTIFICATION:		*HELIOS LIBRARY: 11 APR 1996*
• NO OF DATA IN LIBRARY (I0):		5750412
• SIZE OF RSTAB1 (I1):		535
• SIZE OF RESDAT (I2):		676
• SIZE OF RSTAB2 (I3):		2248
• SIZE OF XSTEMP AND ISTART (I4):		601
• SIZE OF XSBAFR (I5):		0
• MAX SIZE OF XSDATA IN ANY GROUP (I7):	21041	
• SIZE OF P1TEMP (I8):		45
• MAX SIZE OF P1DATA IN ANY GROUP (I10):	12330	
• START INDEX OF RESONANCE DATA (INDRES):		3962
• START INDEX OF XS-DATA (INDXSD):		377688
• START INDEX OF BURNUP DATA (INDBUR):	2498988	
• START INDEX OF P1-MATRICES (INDP1M):		25046160
• NO OF GROUPS (NOG):		190
• NO OF FAST GROUPS (NOFG):		63
• NO OF RESONANCE GROUPS (NORG):	69	
• NO OF THERMAL GROUPS (NOTG):	62	
• NO OF FISSION SOURCE GROUPS (NCHI):		72
• NO OF ISOTOPES INCLUDING BA (NELT):		271
• NO OF ISOTOPES WITH COMPLETE XS SETS (NELR):	169	
• NO OF RESONANCE ISOTOPES (NRES):		31
• NO OF BURNUP ISOTOPES (NBUR):		158
• NO OF BURNABLE ABSORBER ISOTOPES (NBA):		0
• NO OF ISOTOPES WITH FISSION XS (NFIS):		32
• NO OF ISOTOPES WITH P1-DATA (NP1):		19
• NO OF ISOTOPES WITH FISSION SPECTRA (NCHIX):		13
• NO OF ISOTOPES WITH N2N-DATA (N2N):		4
• NO OF ISOTOPES WITH N3N-DATA (N3N):		4

- NO OF GAMMA DATA (IG0, INCLUDED IN I0): 2850512
- SIZE OF TEMPNG AND NGSTAR (IG1): 460
- MAX NO OF GAMMA XS IN ANY GROUP (IG2): 2982
- MAX NO OF G-PROD XS IN ANY GROUP (IG3): 71984
- START INDEX OF GAMMA DATA, SAVED AT I0: 2899899
- NO OF GAMMA GROUPS (NOGG): 48
- NO OF GAMMA ISOTOPES (NGISO): 270

4.3. Materials

In the generalized model of the Saxton reactor core, a limited number of different material compositions were defined in order to simulate the real system. These material compositions have been checked for consistency with information available in the applied nuclear data library. A complete list of different compositions used and their characteristics are listed below together with corresponded nomenclature:

- Fuel compositions:
 - UOX
 - MOXP
 - MOXV
- Non-fuel compositions:
 - SS304
 - ZR40
 - INCONEL
 - AL
 - MCR
 - H2O

Fuel compositions:

Composition name: UOX
 Application: fuel element (type: 001; fuel region)
 Description:

Component	Symbol	HELIOS Library ID	Atomic density, b ⁻¹ cm ⁻¹
Uranium	²³⁵ U	92235	1.303954E-3
	²³⁸ U	92238	2.141301E-2
Oxygen	O	8016	4.543393E-2

Composition name: MOXP
 Application: fuel element (type: 002, 005; fuel region)
 Description:

Component	Symbol	HELIOS Library ID	Atomic density, b ⁻¹ cm ⁻¹
Plutonium	²³⁹ Pu	94239	1.376016E-03
	²⁴⁰ Pu	94240	1.303178E-04
	²⁴¹ Pu	94241	1.353359E-05
	²⁴² Pu	94242	6.082511E-07
Uranium	²³⁵ U	92235	1.540774E-04
	²³⁸ U	92238	2.136511E-02
Oxygen	O	8016	4.607963E-02

Composition name: MOXV
 Application: fuel element (type: 003, 004; fuel region)
 Description:

Component	Symbol	HELIOS Library ID	Atomic density, $b^{-1}cm^{-1}$
Plutonium	²³⁹ Pu	94239	1.273547E-03
	²⁴⁰ Pu	94240	1.206133E-04
	²⁴¹ Pu	94241	1.252577E-05
	²⁴² Pu	94242	5.629558E-07
Uranium	²³⁵ U	92235	1.426035E-04
	²³⁸ U	92238	1.977409E-02
Oxygen	O	8016	4.264817E-02

Non-fuel compositions:

Composition name: SS304
 Application: fuel element (type: 001, 004, 005; clad region); grid material
 Description:

Component	Symbol	HELIOS Library ID	Atom Density [#/(bn*cm)]
Iron	Fe	26054	3.45429E-03
		26056	5.36981E-02
		26057	1.22943E-03
		26058	1.63659E-04
Chromium	Cr	24050	7.59235E-04
		24052	1.46401E-02
		24053	1.65996E-03
		24054	4.13162E-04
Nickel	Ni	28058	5.28420E-03
		28060	2.02018E-03
		28061	8.74489E-05
		28062	2.77853E-04
		28064	7.04171E-05
Manganese	Mn	25055	1.74072E-03
Silicon	Si	14000	1.57021E-03
			7.95046E-05
			5.27710E-05
Carbon	C	6000	3.14975E-04
			3.50363E-06
Phosphorus	P	15031	6.94688E-05

Composition name: ZR40
 Application: fuel element (type: 002, 003; clad region)
 Description:

Component	Symbol	HELIOS Library ID	Atomic density, $b^{-1}cm^{-1}$
Zirconium	Zr	40000	4.259454E-02
Tin	Sn	50112	See Table 2b
		50114	
		50115	
		50116	
		50117	
		50118	
		50119	
		50120	
		50122	
Iron	Fe	26054	See Table 2b
		26056	
		26057	
		26058	

Chromium	Cr	24050 24052 24053 24054	See Table 2b
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Composition name: INCONEL
 Application: grid material
 Description:

Component	Symbol	HELIOS Library ID	Atomic density, $b^{-1}cm^{-1}$
Iron	Fe	26054	See Sec. 3.3
		26056	
		26057	
		26058	
Chromium	Cr	24050	See Sec. 3.3
		24052	
		24053	
		24054	
Nickel	Ni	28058	See Sec. 3.3
		28060	
		28061	
		28062	
		28064	
Silicon	Si	14000	See Sec. 3.3
Titanium	Ti	22000	See Sec. 3.3

Composition name: AL
 Application: grid material
 Description:

Component	Symbol	HELIOS Library ID	Atomic density, $b^{-1}cm^{-1}$
Aluminum	Al	13027	See Sec. 3.3

Composition name: MCR
 Application: control rod material
 Description:

Component	Symbol	HELIOS Library ID	Atomic density, $b^{-1}cm^{-1}$
Silver	Ag	47107	See Sec. 3.4
		47109	
Indium	In	49113	See Sec. 3.4
		49115	
Cadmium	Cd	48000	See Sec. 3.4

Composition name: H2O
 Application: coolant material
 Description:

Component	Symbol	ID	Atomic density, $b^{-1}cm^{-1}$
Hydrogen	H	1006	See Sec. 3.5
Oxygen	O	8001	See Sec. 3.5

4.4. Circular cylindrical system

The structure of an elementary unit cell is shown in Figure 11 below. This unit cell is centered on one fuel element. The fuel region has been subdivided in six concentric internal regions. The influence of the number of concentric regions on computations was investigated.

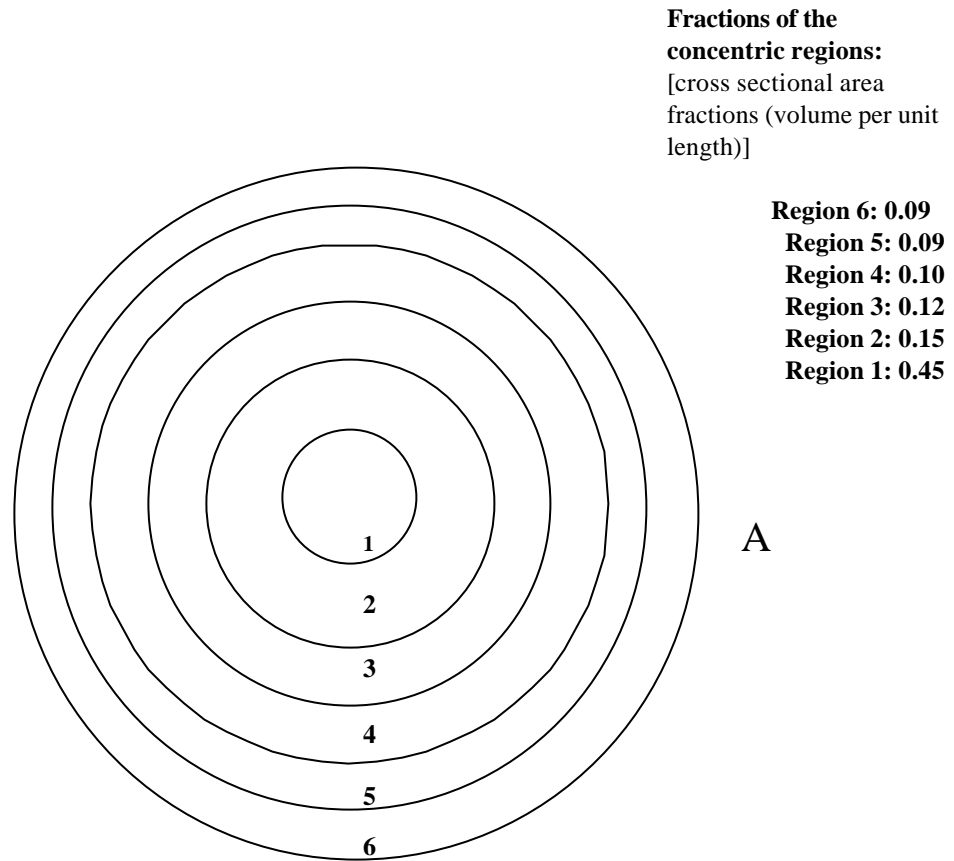


Figure 11. Elementary unit cell centered on one fuel element:
A – moderator (coolant)
6 – clad (zirconium or stainless steel)
1-5 – fuel regions; initially all same material

4.5. Structure description

The fuel loadings for Saxton Cores I, II and III are shown in Fig. 12 A, 12 B and 12C. The control rod locations are shown in Fig. 13. Models of the twenty-one fuel assemblies and one sub-assembly follow these figures.

B1							F1
		C1, 71 (001,700) UOX SS304 SS304		D1, 71 (001, 300) UOX SS304 SS304		E1, 49 (001, 800) UOX SS304 SS304	
	B2, 72 (001, 100) UOX SS304 SS304	C2, 72 (001, 100) UOX SS304 SS304		D2, 72 (001, 100) UOX SS304 SS304		E2, 71 (001, 700) UOX SS304 SS304	F2, 72 (001, 100) UOX SS304 SS304
	B3, 61 (001, 200) UOX SS304 SS304	C3, 70 (001, 500) UOX SS304 SS304		D3, 61 (001, 200) UOX SS304 SS304		E3, 71 (001, 300) UOX SS304 SS304	F3, 71 (001, 700) UOX SS304 SS304
	B4, 71 (001, 300) UOX SS304 SS304	C4, 71 (001, 100) UOX SS304 SS304		D4, 70 (001, 500) UOX SS304 SS304		E4, 72 (001, 100) UOX SS304 SS304	F4, 61 (001, 600) UOX SS304 SS304
B5							F5
		C5, 61 (001, 400) UOX SS304 SS304		D5, 71 (001, 300) UOX SS304 SS304		E5, 72 (001, 100) UOX SS304 SS304	

Simplified image of each fuel assembly contains the following information:

- Assembly location (1C, 1D, etc.)
- Number of fuel rods per assembly (72, etc.)
- Fuel element type (001, 002, etc.) according to description made above.
- Clad material (SS304, ZR40, etc.)
- Model type of fuel assembly geometry (100, 200, etc.) according to description made above.
- Assembly grid type (SS304, INCONEL, etc.)

Figure 12A: Saxton reactor core model (Saxton uranium core, Saxton Core I configuration)

1B								1F
		C1, 71 (001, 700) UOX SS304 SS304 503-10-6		D1, 71 (001, 300) UOX SS304 SS304 503-1-7		E1, 49 (001, 800) UOX SS304 SS304 503-7-1		
	B2, 72 (001, 100) UOX SS304 SS304 503-10-2	C2, 72 (002, 100) MOXP ZR40 INCONEL 503-12-2		D2, 72 (003, 100) MOXV ZR40 INCONEL 503-12-5		E2, 71 (002, 700) MOXP ZR40 INCONEL 503-12-4		F2, 72 (001, 100) UOX SS304 SS304 503-10-3
	B3, 61 (001, 200) UOX SS304 SS304 503-2-3	C3, 70 (002, 005, 500) MOXP ZR40/SS304 INCONEL 503-12-3		D3, 61 (002, 004, 005, 200) MOXP/V ZR40/SS304 INCONEL 503-13-1		E3, 71 (002, 300) MOXP ZR40 INCONEL 503-12-6		F3, 71 (001, 700) UOX SS304 SS304 503-1-19
	B4, 71 (001, 300) UOX SS304 SS304 503-10-4	C4, 71 (002, 100) MOXP ZR40 INCONEL 503-12-7		D4, 70 (003, 004, 500) MOXV ZR40/SS304 INCONEL 503-12-1		E4, 72 (002, 100) MOXP ZR40 INCONEL 503-12-8		F4, 61 (001, 600) UOX SS304 SS304 503-1-11
B5								F5
		C5, 61 (001, 400) UOX SS304 SS304 503-11-2		D5, 71 (001, 300) UOX SS304 SS304 503-1-10		E5, 72 (001, 100) UOX SS304 SS304 503-10-5		

Simplified image of each fuel assembly contains the following information:

- Assembly location (1C, 1D, etc.)
- Number of fuel rods per assembly (72, etc.)
- Fuel element type (001, 002, etc.) according to description made above
- Model type of fuel assembly geometry (100, 200, etc.) according to description made above
- Type of fuel (UOX, MOXP, MOXV etc)
- Clad material (SS304, ZR40, etc.)
- Assembly grid type (SS304, INCONEL, etc.)
- Assembly serial number (prototype assembly ID (Saxton project reports), 503-13-1, etc.)

Figure 12B: Saxton reactor core model (Saxton partial plutonium core, Saxton core II configuration)

B1							F1
			C1, 71 (001, 700) UOX SS304 SS304 503-10-6		D1, 71 (001, 300) UOX SS304 SS304 503-1-7		E1, 49 (001, 800) UOX SS304 SS304 503-7-1
	B2, 72 (001, 100) UOX SS304 SS304 503-10-2		C2, 72 (002, 100) MOXP ZR40 INCONEL 503-12-2		D2, 72 (003, 100) MOXP ZR40 INCONEL 503-12-5		E2, 71 (002, 700) MOXP ZR40 INCONEL 503-12-4
	B3, 61 (001, 200) UOX SS304 SS304 503-2-3		C3, 70 (001, 500) UOX 12.5% ²³⁵ U ZR40/SS304 INCONEL 503-12-3		D3, 61 (002, 004, 005, 200) MOXP/V ZR40/SS304 INCONEL 503-13-1		E3, 71 (001, 300) UOX 12.5% ²³⁵ U ZR40 INCONEL 503-12-6
	B4, 71 (001, 300) UOX SS304 SS304 503-10-4		C4, 71 (002, 100) MOXP ZR40 INCONEL 503-12-7		D4, 70 (003, 004, 500) MOXP ZR40/SS304 INCONEL 503-12-1		E4, 72 (002, 100) MOXP ZR40 INCONEL 503-12-8
B5							F5
			C5, 61 (001, 400) UOX SS304 SS304 503-11-2		D5, 71 (001, 300) UOX SS304 SS304 503-1-10		E5, 72 (001, 100) UOX SS304 SS304 503-10-5

Simplified image of each fuel assembly contains the following information:

- Assembly location (C1, D1, etc.)
- Number of fuel rods per assembly (72, etc.)
- Assembly serial number (prototype assembly ID (Saxton project reports), 503-13-1, etc.)
- Fuel element type (001, 002, etc.) according to description made above (see Fig. 2 for summary)
- Clad material (304-SS, ZR40, etc.)
- Model type of fuel assembly geometry (100, 200, etc.) according to description made above (see Fig. 3 for summary)
- Assembly grid type (304-SS, INCONEL, etc.)

Figure 12C: Saxton reactor core model (Saxton partial plutonium core, Saxton core III configuration) Saxton Core III Fuel Loading at BOL

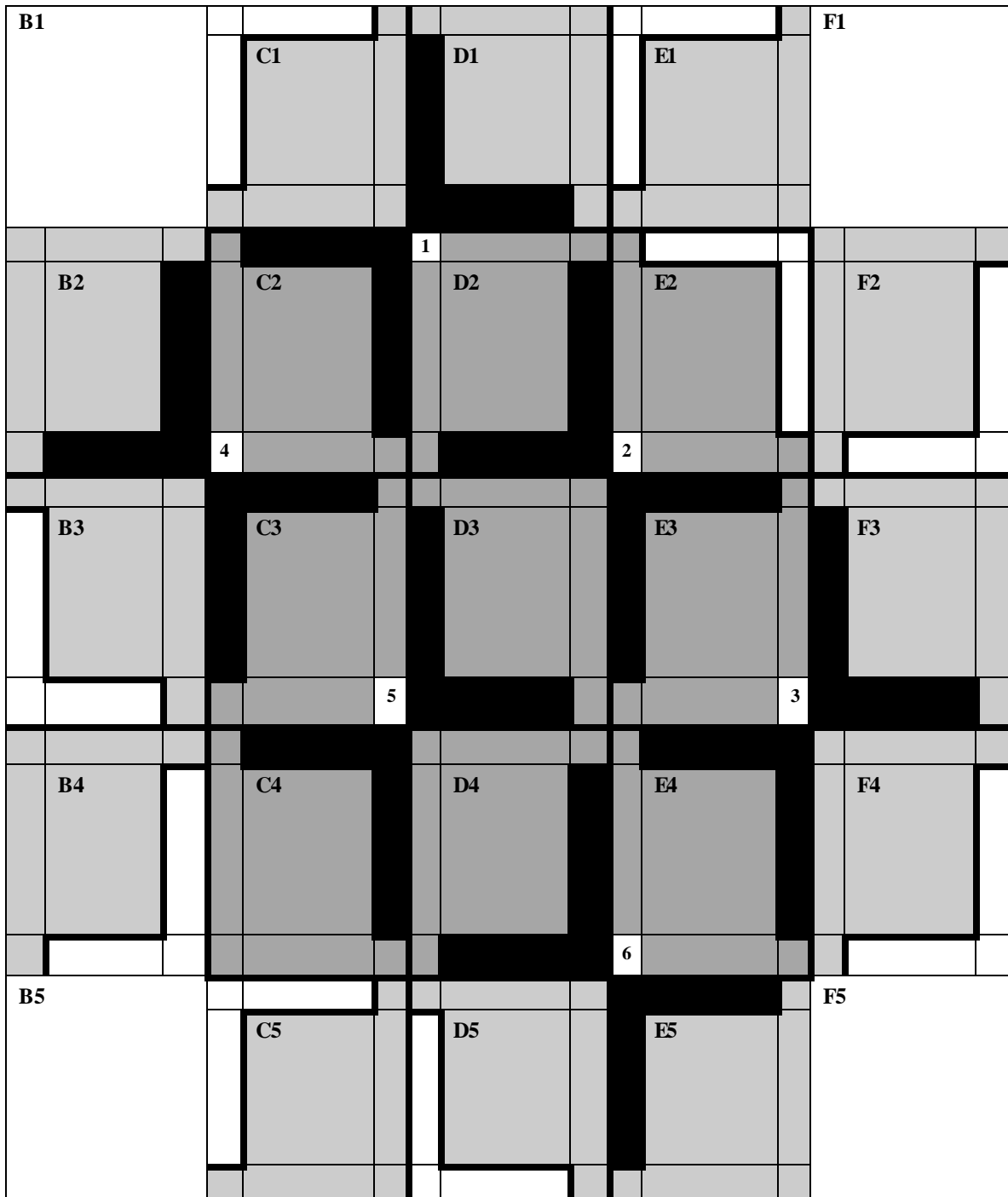


Figure 13: Location and numbering of the control rods in the Saxton reactor core model

The following assembly maps are the detailed assemblies of Core II.

Assembly: **Uranium assembly**
 Assembly location: B2
 Assembly type: 100
 Prototype assembly ID (Saxton project reports): 503-10-2

36	09	18	27	36	45	54	63	72	81
35	08	17	26	35	44	53	62	71	80
34	07	16	25	34	43	52	61	70	79
33	06	15	24	33	42	51	60	69	78
32	05	14	23	32	41	50	59	68	77
31	04	13	22	31	40	49	58	67	76
30	03	12	21	30	39	48	57	66	75
29	02	11	20	29	38	47	56	65	74
28	01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 72
- Rods:
 - 304-SS UO₂ Pelletized Fuel (type 001)

Assembly:

Assembly location:

Assembly type:

Prototype assembly ID (Saxton project reports):

Uranium assembly

B3

200

503-2-3

27	09	18	27	36	45	54	63	72	81
26	08	17	26	35	44	53	62	71	80
25	07	16	25	34	43	52	61	70	79
24	06	15	24	33	42	51	60	69	78
23	05	14	23	32	41	50	59	68	77
22	04	13	22	31	40	49	58	67	76
21	03	12	21	30	39	48	57	66	75
20	02	11	20	29	38	47	56	65	74
19	01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 61
- Rods:
 - 304-SS UO₂ Pelletized Fuel (type 001)
 - 45, 77 – flux thimble

Assembly:

Assembly location:

Assembly type:

Prototype assembly ID (Saxton project reports):

Uranium assembly

B4

300

503-10-4

18	09	18	27	36	45	54	63	72	81
17	08	17	26	35	44	53	62	71	80
16	07	16	25	34	43	52	61	70	79
15	06	15	24	33	42	51	60	69	78
14	05	14	23	32	41	50	59	68	77
13	04	13	22	31	40	49	58	67	76
12	03	12	21	30	39	48	57	66	75
11	02	11	20	29	38	47	56	65	74
10	01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 71
- Rods:
 - 304-SS UO₂ Pelletized Fuel (type 001)
 - 45 – flux thimble

Assembly:

Assembly location:

Assembly type:

Prototype assembly ID (Saxton project reports):

Uranium assembly

C1

700

503-10-6

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 71
- Rods:
 - 304-SS UO₂ Pelletized Fuel (type 001)
 - 77 – flux thimble

Assembly:

Assembly location:

Assembly type:

Prototype assembly ID (Saxton project reports):

Plutonium assembly

C2

100

503-12-2

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 72
- Rods:

Zircaloy-4 Clad PuO₂-UO₂ Pelletized Fuel (type 002)
05, 37 – removable rods

Assembly:

Assembly location:

Assembly type:

Prototype assembly ID (Saxton project reports):

Plutonium assembly

C3

500

503-12-3

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 70
- Rods:

11, 12, 22, 23, 24, 25, 34, 43, 52, 58, 59, 60, 61, 62, 65, 66, 70, 71 – 304-SS Clad PuO₂-UO₂ Pelletized Fuel (type: 005)

Zircaloy-4 Clad PuO₂-UO₂ Pelletized Fuel (type 002)

77 – flux thimble

37 – secondary source rod

Assembly:

Assembly location:

Assembly type:

Prototype assembly ID (Saxton project reports):

Plutonium assembly

C4

100

503-12-7

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 71
- Rods:

Zircaloy-4 Clad PuO₂-UO₂ Pelletized Fuel (type: 002)

37 – flux thimble

05 – removable rod

Assembly:**Uranium assembly**

Assembly location:

C5

Assembly type:

400

Prototype assembly ID (Saxton project reports):

503-11-2

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 61
- Rods:
 - 304-SS Clad UO₂ Pelletized Fuel (type: 001)
 - 77 – flux thimble

Assembly:**Uranium assembly**

Assembly location:

D1

Assembly type:

300

Prototype assembly ID (Saxton project reports):

503-1-7

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 71
- Rods:
 - 304-SS Clad UO₂ Pelletized Fuel (type: 001)
 - 77 – flux thimble

Assembly:

Assembly location:

Assembly type:

Prototype assembly ID (Saxton project reports):

Plutonium assembly

D2

100

503-12-5

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 72
- Rods:

Zircaloy-4 Clad PuO₂-UO₂ VIPAC Fuel (type: 003)
05, 45 – removable rods

Assembly:

Assembly location:

Assembly type:

Prototype assembly ID (Saxton project reports):

Plutonium assembly

D3

200

503-13-1

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 61
- Rods:

Zircaloy-4 Clad PuO₂-UO₂ Pelletized Fuel (type: 002)
 07, 63 – 304-SS Clad PuO₂-UO₂ VIPAC Fuel (type: 004)
 64, 79 – 304-SS Clad PuO₂-UO₂ Pelletized Fuel (type: 005)
 45, 77 – flux thimble

Sub-Assembly:**Plutonium sub-assembly**

Sub-Assembly location:

D3

Prototype assembly ID (Saxton project reports):

503-4-26

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 61
- Rods:
 - 33, 49 – Zircalloy-4 Clad PuO₂-UO₂ Pelletized Fuel (type: 002)
 - 42, 40 – removable rods: Zircalloy-4 Clad PuO₂-UO₂ Pelletized Fuel (type: 002)
 - 31, 51 - Zircalloy-4 Clad PuO₂-UO₂ VIPAC Fuel (type: 003)
 - 32, 50 – removable rods: Zircalloy-4 Clad PuO₂-UO₂ VIPAC Fuel (type: 003)
 - 41 – flux thimble

Assembly:

Assembly location:

Assembly type:

Prototype assembly ID (Saxton project reports):

Plutonium assembly

D4

500

503-12-1

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 70
- Rods:

Zircaloy-4 Clad PuO₂-UO₂ VIPAC Fuel (type: 003)

05, 45 – flux thimble

12, 16, 17, 20, 23, 26, 62, 71 – SS304 Clad PuO₂-UO₂ VIPAC Fuel (type: 004)

Assembly:**Uranium assembly**

Assembly location:

D5

Assembly type:

300

Prototype assembly ID (Saxton project reports):

503-1-10

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 71
- Rods:
 - 304-SS Clad UO₂ Pelletized Fuel (type: 001)
 - 77 – flux thimble

Assembly:**Uranium assembly**

Assembly location:

E1

Assembly type:

800

Prototype assembly ID (Saxton project reports):

503-7-1

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 49
- Rods:
 - 304-SS Clad UO₂ Pelletized Fuel (type: 001)
 - 77 – flux thimble

Assembly:

Assembly location:

Assembly type:

Prototype assembly ID (Saxton project reports):

Plutonium assembly

E2

700

503-12-4

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 71
- Rods:

Zircaloy-4 Clad PuO₂-UO₂ Pelletized Fuel (type: 002)

05 – removable rod

37 – flux thimble

Assembly:

Assembly location:

Assembly type:

Prototype assembly ID (Saxton project reports):

Plutonium assembly

E3

300

503-12-6

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 71
- Rods:

Zircaloy-4 Clad PuO₂-UO₂ Pelletized Fuel (type: 002)

77 – removable rod

37 – secondary source rod

Assembly:

Assembly location:

Assembly type:

Prototype assembly ID (Saxton project reports):

Plutonium assembly

E4

100

503-12-8

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 72
- Rods:

Zircaloy-4 Clad PuO₂-UO₂ Pelletized Fuel (type: 002)
05, 37 – removable rods

Assembly:

Assembly location:

Assembly type:

Prototype assembly ID (Saxton project reports):

Uranium assembly

E5

100

503-10-5

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 72
- Rods:

304-SS Clad UO₂ Pelletized Fuel (type: 001)

Assembly:**Uranium assembly**

Assembly location:

F2

Assembly type:

100

Prototype assembly ID (Saxton project reports):

503-10-3

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 72
- Rods:

304-SS Clad UO₂ Pelletized Fuel (type: 001)

Assembly:**Uranium assembly**

Assembly location:

F3

Assembly type:

700

Prototype assembly ID (Saxton project reports):

503-1-19

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 71
- Rods:
 - 304-SS Clad UO₂ Pelletized Fuel (type: 001)
 - 45 – flux thimble

Assembly:**Uranium assembly**

Assembly location:

F4

Assembly type:

600

Prototype assembly ID (Saxton project reports):

503-1-11

09	18	27	36	45	54	63	72	81
08	17	26	35	44	53	62	71	80
07	16	25	34	43	52	61	70	79
06	15	24	33	42	51	60	69	78
05	14	23	32	41	50	59	68	77
04	13	22	31	40	49	58	67	76
03	12	21	30	39	48	57	66	75
02	11	20	29	38	47	56	65	74
01	10	19	28	37	46	55	64	73

Description:

- Total number of fuel rods: 61
- Rods:
 - 304-SS Clad UO₂ Pelletized Fuel (type: 001)
 - 45 – flux thimble

4.6. Resonance treatment

The energy dependence of the cross-sections of the HELIOS nuclear data library has been discretized by dividing the energy range of interest, from 20 MeV to 2.540E-04 eV, into a number of broad groups. These cross-sections have been obtained by flux-averaging point cross-sections (more than 100000) with typical reactor spectra. We have used the RES-option of the RUN operator in HELIOS so that different nuclides have been arranged in the same resonance set. We listed below the resonance sets we elected to use:

- ^{238}U – category 1
- Rest of heavy metal nuclides – category 2
- Non-heavy metal nuclides – category 3

5. Measured Parameters and Model Sensitivity Studies

5.1. Characteristic core calculations with HELIOS

A characteristic HELIOS output without burn-up calculations yields:

```
*****  
* Normal End,      No warning messages issued *  
*  
* Highest memory required by this run: *  
* 735842 data words in module 2 ( 9%) *  
* 18321 char words in module 4 ( 3%) *  
* Total CPU time used = 5.00 minutes *  
*****
```

A typical runtime of HELIOS lasted about 9 hours for the most complicated configurations. Every case was computed with a burnup step equal to 500 MWd. Computations were performed without any decoupling schemes (current) in order to obtain a reference solution.

5.2. Sensitivity analysis

Within the framework of this study, some sensitivity analyses were performed in order to estimate the influence of some model characteristics on the K_{eff} of the system. These sensitivity analyses were needed because some of this information was not properly reported in the series of reports of this program. The following characteristics were varied:

- Material of fuel assembly grid
- Structure of the circular cylindrical system
- L-shaped sections containing fuel
- Temperature distribution within fuel rod (see Fig. 11)
- Structure of the control rod

Results of the sensitivity analysis are summarized below. They show changes in the boron endpoint concentrations with changes in the various characteristics. Note that these results are only for the BOL state where there is no fuel burnup and hence, no burnup distribution in the fuel. It is therefore, not too surprising that the boron concentration is not very sensitive to the number of fuel regions. An interesting comparison would be that of the EOL boron concentration with the different number of fuel regions. However, in that case, due to burnup, the distribution of the fuel will depend on radius and obscure the effect being studied - the sensitivity of the boron concentration on the number of regions in the fuel pins. Thus, the EOL comparison was intentionally avoided and is not given in the report because of several reasons:

- (a) the idea of the provided comparison was to demonstrate necessity to describe fuel pins as several concentric regions even for BOL state (results show that boron concentration is sensitive to the model), and
- (b) it was recognized that comparison for EOL state would show much more significant sensitivity on the fuel pin model, but in the case of EOL state, other factors will influence this sensitivity effect like presence of burnup distribution due to multiple region representation.

This would expand analysis considerably and make it much more complex. As a result, the EOL state was left beyond the scope of the report.

As a result of the sensitivity studies, the following features were used in the calculational model:

- Grid materials as shown in Fig. 12.
- L-shaped sections included or replaced by water according to core configuration.
- 6 concentric regions as shown in Fig. 11.
- Temperature distribution as given below.
- Model B for Control rod as shown below.

In the references, it is mentioned that the control rods had fuel-followed sections but very few details are given as regards to how they were made. Therefore, Model B was utilized.

Conditions:

Model: Unified Saxton reactor core description
 Configuration: Saxton reactor core II, all control rods out, BOL
 Hot conditions: Thermal power, MWt - 23.5; Temperature, K - 810.928 (fuel); 591.483 (ZR40); 588.706 (SS304); 588.706 (INCONEL, AL); 551.483 (H₂O).

Material of fuel assembly grid

Boron endpoint concentration (ppm)

Grid materials	Boron concentration, ppm	Ratio*
1. SS304 and INCONEL	2398.631	1.000
2. SS304 only	2378.431	0.992
3. INCONEL only	2456.953	1.024
4. AL	2542.324	1.059

*ratio: [1, 2, 3, 4]/[1]

Structure of the circular cylindrical system

Boron endpoint concentration (ppm)

Number of concentric regions (see Fig. 11):	Boron concentration, ppm	ratio*
1. 8 regions	2398.631	1.000000
2. 6 regions	2397.395	0.999485
3. 1 region	2396.409	0.999074

*ratio: [1, 2, 3]/[1]

L-shaped sections containing fuel

Boron endpoint concentration (ppm)

Number of concentric regions (see Fig. 11):	Boron concentration, ppm	ratio*
1. Sections installed	2401.714	1.000000
2. Section places treated as water regions	2398.631	0.998716

*ratio: [1, 2]/[1]

Temperature distribution within fuel rod (see fig. 11)

Average fuel temperature, K: 810.928

Special temperatures for concentric regions (6 region scheme), K:

Region 1 (fraction – 0.45):	1033.229
Region 2 (fraction – 0.15):	986.264
Region 3 (fraction – 0.12):	892.334
Region 4 (fraction – 0.10):	779.618
Region 5 (fraction – 0.09):	657.509
Region 6 (fraction – 0.09):	516.614

Averaged over 6 regions fuel temperature, K: **810.928**

Boron endpoint concentration (ppm)

Number of concentric regions (see Fig. 11):	Boron concentration, ppm	ratio*
1. 6 regions	2397.395	1.000000
2. 1 region (average temperature)	2429.547	1.013411

*ratio: [1, 2]/[1]

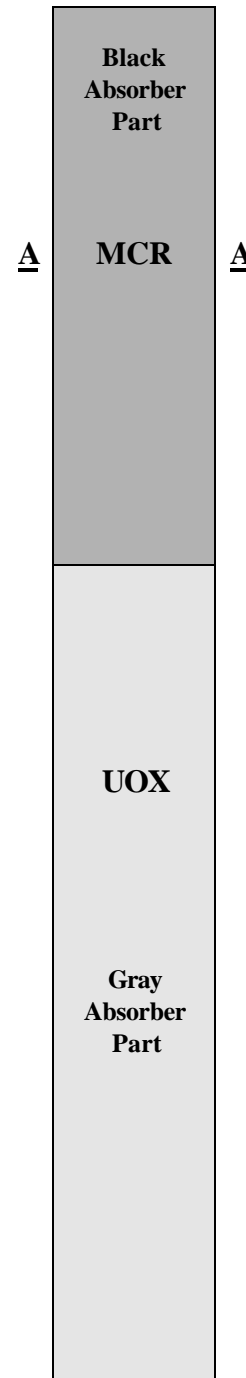
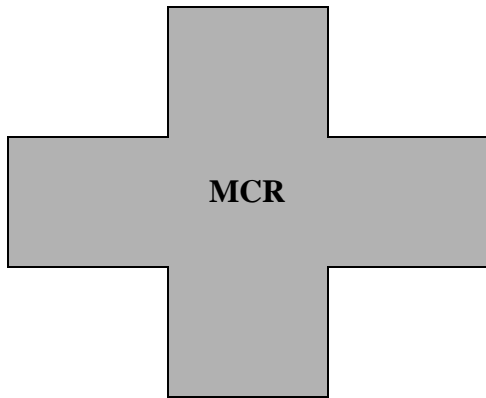
Structure of the control rod

As it was already described above, the reactor used six offset cruciform shaped control rods made of an alloy of silver, indium and cadmium. In the references, it is mentioned that the control rods had fuel followed sections but very few details are given as regards to how they were made. Hence, within the frame of our analysis we consider two models for the description of control rods in the Unified Saxton Reactor Core Model:

Model A:

Control rod cross section:

A-A

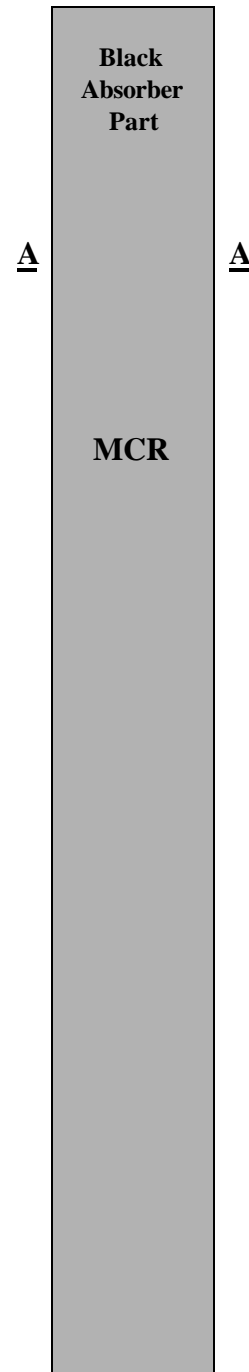
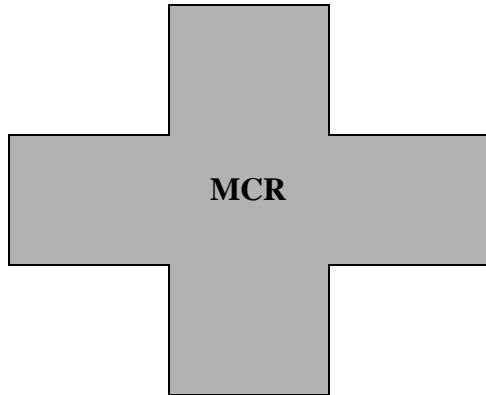


Conditions:	
<p>Rod in:</p> <p>Black Absorber Part (MCR) in the reactor core</p>	<p>Rod out:</p> <p>Gray Absorber Part (UOX) in the reactor core (see Table 2b for composition of UOX)</p>

Model B:

Control rod cross section:

A-A



Conditions:	
Rod in: Black Absorber Part (MCR) in the reactor core	Rod out: Space reserved for control rod is filled with water (H ₂ O composition)

Sensitivity analysis of the control rod model influence on the calculated boron endpoint concentrations (ppm):

Core configuration	Saxton Plutonium Project reports	HELIOS calculations				Burnup, MWD/MTM
		Model A for control rod	ratio*	Model B for control rod	Difference *	
All rods out	2296	2403.912	1.047	2398.631	1.045	BOL
Rods 1, 3, 4,5 & 6 out, Rod 2 in	1702	1785.398	1.049	1780.803	1.046	
Rods 1, 3, 4 & 6 out, Rod 2 & 5 in	1047	1102.491	1.053	1101.902	1.052	
All rods out	1565	1641.685	1.049	1636.900	1.046	3716
Rods 1, 3, 4,5 & 6 out, Rod 2 in	1037	1089.887	1.051	1086.879	1.048	
All rods out	1266	1328.034	1.049	1326.388	1.048	5715
Rods 1, 3, 4,5 & 6 out, Rod 2 in	796	839.78	1.055	837.153	1.052	
All rods out	683	719.882	1.054	718.492	1.052	10490
Rods 1, 3, 4,5 & 6 out, Rod 2 in	312	329.784	1.057	328.573	1.053	

*ratio: [Model A, Model B]/[Report]

On the basis of these sensitivity analyses, we reached the following conclusions:

- There is constant discrepancy between the computational HELIOS results and experimental results; HELIOS calculations typically overpredict the Saxton characteristics;
- HELIOS results overpredict boron concentrations except when using type 304-SS fuel assembly grid structure;
- Sensitivity studies were performed in order to evaluate the influence of the control-rod model on results. Both models overpredict boron concentration when compared to available experimental information.

Investigation of certain calculational and experimental details were not considered due to funding limitations. Specific notes on the computer models follow:

- Critical boron concentrations were determined by a manual search.
- The calculations were performed at different boron concentrations and the results interpolated to obtain the boron concentration that corresponds to a k_{eff} of unity. (HELIOS does not have an option to perform such criticality searches.)
- According to the Core II description, assembly E-1 is from Core I. E-1 was depleted using the Core I model.
- Though no information on the axial buckling values used in the 2-D HELIOS calculations; several approaches have been studied: zero axial bucklings and bucklings calculated using actual height of the core.
- Though no information on the boron concentrations during the fuel burnup is presented, the SAXTON materials core histories were followed whenever information was found. Power generated in the followers was not included.
- Actual histories of the SAXTON core irradiation were described with very limited amount of information. As a result, only MOL Core III was modeled.
- More results would be useful even if there are not corresponding measurements (boron concentrations or k_{eff} vs. burnup for each core, power distributions at various burnup points, fuel isotopics at various burnup points, rod worths, etc.) The studies did not go in many details and many aspects due to time constraints of the project.

5.3. Saxton reactor Core I

Core I was used as a reference core configuration for the verification of the generalized model of the core geometry. Computations performed during the Saxton project are provided in Table 3.

Table 3. Some characteristics of the Saxton reactor Core I that were calculated with HELIOS and their comparison with calculated results obtained during the Saxton Plutonium Project

Characteristics	Saxton Plutonium Project reports (Computed, measured not available)
Effective multiplication factor at BOL conditions: <ul style="list-style-type: none"> • Zero power, all control rods out, cold conditions (293.1500 K) • Zero power, all control rods out, hot conditions (549.8167 K) • Power (20 MWt), all control rods out, hot conditions (549.8167 K) 	 1.241* 1.171* 1.156*
Boron concentration (natural boron), ppm: <ul style="list-style-type: none"> • To shut down the reactor down to $K_{eff}=0.97$ (all control rods out, zero power): <ul style="list-style-type: none"> Cold (293.1500 K) Hot (549.8167 K) • To control at power (20 MW, all control rods out, hot conditions (549.8167 K)) • Approximate concentration for worth of 1%: <ul style="list-style-type: none"> Cold (293.1500 K) Hot (549.8167 K) 	 2200 2330 1835 80 110
Estimated control rod worth (average per rod), %	3.8

*Computational results, LEOPARD-PDQ computer code system (Saxton Plutonium Project report)

5.4. Saxton reactor Core II

Saxton Core II achieved criticality on 12/06/65 and started initial power operation on 12/29/65. After operating at initial power levels of 17 MW and 21 MW, it achieved its rated power of 23.5 MW on 02/07/66. From Jan 68 onward, the reactor was operated at an escalated power of 30.6 MW. Control rod 2 was banked to 50.8cm (20 inches). The maximum linear power under these conditions in the mixed oxide fuel was estimated to be 0.60696 kW/cm (18.5 kW/ft). The reactor was operated at this power level until the end of life (EOL).

At the beginning of core life (BOL) and periodically throughout the core life, measurements were made to determine the control rod worth, boron worth and moderator temperature coefficients. A summary of experimental results is available in the Saxton Plutonium Project reports:

- Zero Power BOL Measurements at Ambient Temperature (boron endpoint measurement for all rods out, for rod 2 in, for rod 2 and 5 in, and for rods 2 and 5 in with rods 1 and 6 partially inserted, were made with the reactor at cold conditions);
- Moderator Temperature Coefficient Measurements (experimental determination of differential temperature coefficients at each of the endpoint boron concentrations in the cold condition of the core);
- Cold-to-Hot Rod Reactivity Defects (the reactivity effect associated with heatup/cooldown was measured at two boron concentrations);
- Zero Power Reactivity Measurements at Operating Temperatures (at the beginning of core life and periodically throughout the cycle, measurements were made for all-rods-out and for the following quantities with rod 2 inserted: the boron concentration, the control rod worth for rod 2, the differential boron worth, the differential moderator temperature coefficient, and the xenon worth);

Some experimental results are summarized in Table 4.

Table 4. Saxton reactor Core II experimental information from the Saxton Plutonium Project reports

A. Zero power BOL measurements at certain temperature
(boron endpoint measurement (ppm) for all rods out, rod 2 in, rod 2 and 5 in)

Boron endpoint concentration (ppm)

Core configuration	Core average Temperature, K	Saxton Plutonium Project reports
All rods out	330.372	2718
Rods 1, 3, 4,5 & 6 out Rod 2 in	341.483	2268
Rods 1, 3, 4 & 6 out Rod 2 & 5 in	341.483	1742

**B. Endpoint measurements at hot conditions (Core average temperature: 549.8167 K)
(Boron endpoint measurement (ppm) for all rods out, rod 2 in, rod 2 and 5 in)**

Boron endpoint concentration (ppm)

Core configuration	Saxton Plutonium Project reports	Measured inverse Boron worth (ppm/%)	Burnup, MWD/MTM
All rods out	2296	161	BOL
Rods 1, 3, 4,5 & 6 out, Rod 2 in	1702		
Rods 1, 3, 4 & 6 out, Rod 2 & 5 in	1047		
All rods out	1565	160	3716
Rods 1, 3, 4,5 & 6 out, Rod 2 in	1037		
All rods out	1266	151	5715
Rods 1, 3, 4,5 & 6 out, Rod 2 in	796		
All rods out	683	-	10490
Rods 1, 3, 4,5 & 6 out, Rod 2 in	312		

Four mixed oxide fuel rods (2 pellet and 2 VIPAC type, clad in zircaloy) were irradiated in the peripheral assembly 1E from June 11, 1965 until the end of the year. Operation of this subassembly in the center position commenced on December 28, 1965 and continued until March 31, 1966. Post irradiation evaluation of these fuel rods that had been subjected to average exposure levels of 4500 MWD/MTM and 6100 MWD/MTM peak burnup, was carried out. Though the designed linear power rating was 0.525kW/cm(16 KW/ft), this actual operating linear power during this period was 0.348kW/cm(10.6 KW/ft).

The program for the examination of these rods included:

- (a) External examination (visual and by stereo microscope) of four rods for damage, crud, discoloration, weld area corrosion, etc.
- (b) Dimensional check of four rods for “bambooning” of cladding at pellet ends, for cladding creep, for cladding ovality etc.
- (c) Gamma scan to determine the peak power location
- (d) Collection and analysis of fission gases
- (e) Sectioning of two rods at peak burnup areas (two cuts per rod for one VIPAC and one pelletized rod)
- (f) Metallographic examination of samples from the two cut rods
- (g) Determination of the hydrogen content in the cladding in the two cut rods.
- (h) Analysis for Cs-137, Sr-90 and Pu/U on samples taken from the area of the four cuts referred to in Item (e)
- (i) Mass spectrometric analyses for: U-234, U-235, U-236, U-238, Pu-239, Pu-240, Pu-241, Pu-242, Pu-239/U-238, Nd-148

Results are shown in Table 5A and 5B.

Table 5A. Summary of the Radiochemical Analysis Data

Sample	Cs-137(1)	Sr-90(2)	Pu-239/U-238 X 10 ² (atom ratio)	Nd-148/U-238 X 10 ⁶ (atom ratio)
D-BU-1	4.53	1.78	5.576 ± 0.45	115.0 ± 1.1
D-BU-2	4.07	1.7	5.845 ± 0.52	102.2 ± 0.8
X1-BU-1	3.84	1.46	5.951 ± 0.40	97.42 ± 0.8
X1-BU-2	3.6	1.34	5.842 ± 0.36	87.47 ± 0.67

(1) dpm/gm final Ux10⁻¹⁰ on 06-05-67 (± 6 %) (2) dpm/gm final Ux10⁻¹⁰ on 06-22-67 (± 68%) dpm/gm = disintegrations per minute per gram of uranium (metal) of burned fuel multiplied by 10⁻¹⁰.

REFERENCES: WCAP 3385-10 and WCAP 1871 and WCAP 3385-12)- see figure 14 for more detailed information.

Table 5B. Summary of alpha spectrometric analyses and for Saxton Core II configuration at EOL (October 18, 1968)

Sample taken from assembly: C3
 Fuel rod lattice position: 50 (see picture on Fig. 14)
 Fuel burnup, MWD/MTM: 20450

Characteristics	Saxton Plutonium Project report (alpha spectrometric analyses)
²³⁸ U weight fraction in total U	0.99398
²³⁷ Np/ ²³⁸ U atom ratio (x 10 ⁴)	1.15
²³⁶ Pu/ ²³⁹ Pu atom ratio(x 10 ⁹)	3.86 ± 0.97
²³⁸ Pu/ ²³⁹ Pu atom ratio(x 10 ³)	1.36 ± 0.04
²⁴¹ Am/ ²³⁹ Pu atom ratio (x 10 ²)	0.812 ± 0.191
²⁴² Cm/ ²³⁹ Pu atom ratio (x 10 ⁴)	5.48
²⁴⁴ Cm/ ²³⁹ Pu atom ratio (x 10 ⁴)	1.29

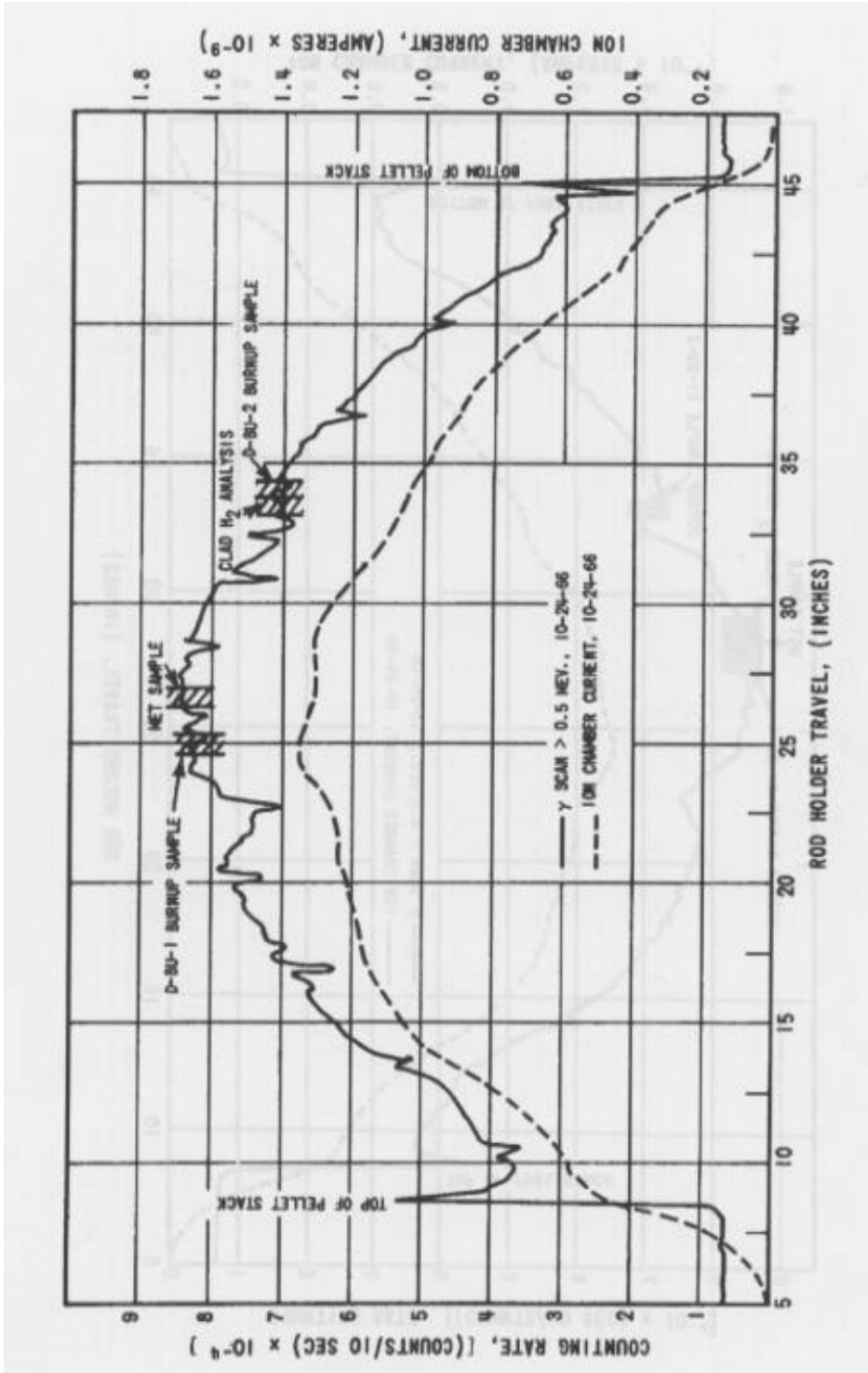


Figure 14: Gamma scan of Saxton Core II rod. Location of the sampling performed for burnup experimental studies

5.5. Saxton reactor Core III

The Core III layout is shown in Fig. 15, and the core relative assembly powers are shown on a core layout in Fig. 16. Original project results are shown in Tables 6 and 7.

Summary of Saxton Core III Operating History Through EOL:

	Through March 31, 1971	During April-May 1972	EOL
Energy Generated, MWd	5352	467	5819
Number of Load Cycles	631	0	631
Peak Linear Power, kW/cm(kW/ft) ^a			
1. Peak Power Rod	0.6955 (21.2)	0.4823 (14.7)	----
2. Peak Burnup Rod (in Center 3x3)	0.4659 (14.2)	0.3248 (9.9)	----
3. Peak Burnup Rod	0.3970 (12.1)	0.2756 (8.4)	----
Peak Pellet Burnup, MWd/MTM ^b			
1. Peak Power Rod	26400	2700	29100
2. Peak Burnup Rod (in Center 3x3)	49200	1800	51000
3. Peak Burnup Rod	45600	1500	47100

(a) Best estimate, thermal basis (thermal=0.974 fission)

(b) Best estimate, fission basis

Saxton Core III power history

Period	Calendar Time	Eff. Full Power Hrs. @ 23.5 MW		Core Energy Production, MWd	Core Burnup, MWd/MTM
		Step	Accumulative		
1	BOL 12/11/69 to 02/12/70	700	700	685	952
2	02/13/70 to 05/14/70	1021	1721	1685	2340
3	08/24/70 to 11/17/70	1000	2721	2664	3700
4	MOL 11/18/70 to 03/12/71	993	3714	3637	5051
5	11/10/71 to 01/31/72	1045	4759	4660	6472
6	EOL 02/01/72 to 05/01/72	1184	5943	5819	8082

The reactor Core III operation phase was successfully completed with the achievement of peak pellet burnups exceeding 50000 MWd/MTM.

B1							F1
		C1, 71 (001, 700)		D1, 71 (001, 300)		E1, 49 (001, 800)	
		(1) 0.56 (2) 0.59		0.60 0.62		0.52 0.52	
	B2, 72 (001, 100)		C2, 72 (002, 100)		D2, 72 (003, 100)		F2, 72 (001, 100)
	0.62 0.59		1.76 1.73		1.81 1.82		0.54 0.55
	B3, 61 (001, 200)		C3, 70 (001, 500)		D3, 61 (002, 004, 005, 200)		F3, 71 (001, 700) UOX
	0.86 0.84		1.90 1.88		2.25 2.20		0.69 0.69
	B4, 71 (001, 300)		C4, 71 (002, 100)		D4, 70 (003, 004, 500)		F4, 61 (001, 600)
	0.62 0.65		1.96 1.93		2.29 2.28		0.69 0.62
B5							F5
		C5, 61 (001, 400)		D5, 71 (001, 300)		E5, 72 (001, 100)	
		0.64 0.63		0.76 0.86		0.51 0.53	

Simplified image of each fuel assembly contains the following information:

- Assembly location (C1, D1, etc.)
- Number of fuel rods per assembly (72, etc.)
- Fuel element type (001, 002, etc.) according to description made above
- Model type of fuel assembly geometry (100, 200, etc..)
- Relative assembly powers:

- (1) Measured pre-MOL (01/71)
- (2) Measured post-MOL (11/71)

Figure 15: Relative Assembly Powers for Saxton reactor Core model (Saxton partial plutonium Core, Saxton Core III configuration)

B1							F1
		C1, 71 (001, 700)		D1, 71 (001, 300)		E1, 49 (001, 800)	
		13.5		12.7		13.0	
	21, 72 (001, 100)		C2, 72 (002, 100)		D2, 72 (003, 100)		F2, 72 (001, 100)
	1.7		2.8		2.2		7.8
	B3, 61 (001, 200)		C3, 70 (001, 500)		D3, 61 (002, 004, 005, 200)		F3, 71 (001, 700)
	2.3		1.6		0.0		1.5
	B4, 71 (001, 300)		C4, 71 (002, 100)		D4, 70 (003, 004, 500)		F4, 61 (001, 600)
	3.2		1.5		1.3		7.5
B5							F5
		C5, 61 (001, 400)		D5, 71 (001, 300)		E5, 72 (001, 100)	
		6.0		6.5		8.6	

Simplified image of each fuel assembly contains the following information:

- Assembly location (C1, D1, etc.)
- Number of fuel rods per assembly (72, etc.)
- Fuel element type (001, 002, etc.) according to description made above
- Model type of fuel assembly geometry (100, 200, etc..)
- Relative assembly powers:

[Measured post-MOL (11/71)-Saxton/LEOPARD/PDQ]/Measured post MOL] X 100

Figure 16: Percent Difference in the Measured and Calculated Relative Assembly Powers for Saxton reactor Core model (Saxton partial plutonium Core, Saxton Core III configuration)

Table 6. Zero power BOL measurements at certain temperature

(boron endpoint measurement (ppm) for all rods out, core average temperature: 549.8167 K)
 Boron endpoint concentration (ppm)

Core configuration	Saxton Plutonium Project reports
Critical boron, ppm	1082

Table 7. Zero power MOL measurements at certain temperature

(boron endpoint measurement (ppm) for all rods out, core average temperature: 549.8167 K)
 Boron endpoint concentration (ppm)

Core configuration	Saxton Plutonium Project reports
Critical boron, ppm	591

Table 8 presents a comparison of the results of the mass spectrometric analyses performed on the two burnup samples from rod RR.

Table 8. Summary of the Radiochemical Analysis Data

Sample Location (From the bottom of the fuel rod) cm (in)	U(Atom %) Abundance				Pu (atom %) Abundance				
	U^{234}	U^{235}	U^{236}	U^{238}	Pu^{238}	Pu^{239}	Pu^{240}	Pu^{241}	Pu^{242}
27.94 - 29.21 (11.0"–11.5")	4.31 -3	0.486	0.0506	99.459	0.315	61.411	28.084	8.627	1.563
46.99 - 48.26 (18.5"–19.0")	4.34 -3	0.476	0.053	99.467	0.346	60.302	28.673	8.984	1.695

Sample Location (From the bottom of the fuel rod) cm (in)	Atom Ratio	
	$Nd^{148}/U^{238} (x 1e4)$	$Pu^{239}/U^{238} (x 1e2)$
27.94 - 29.21 (11.0" – 11.5")	5.943	3.097
46.99 - 48.26 (18.5" – 19.0")	6.062	2.972

REFERENCE: WCAP-3385-26

The mass spectrometric analysis for rod RR has been confirmed independently and the obtained burnup results are:

Sample Location (From the bottom of the fuel rod) cm (in)	Atom Ratio		Burnup, MWd/MTM		
	$Nd^{148}/U^{238} (x 1e4)$	$Pu^{239}/U^{238} (x 1e2)$	Heavy Isotope	Nd^{148}	Percent difference
27.94 - 29.21 (11.0" - 11.5")	5.943	3.097	33690	31990	+5.3
46.99 - 48.26 (18.5" - 19.0")	6.062	2.972	34400	32640	+5.3

The best estimate rod average burnup and peak pellet burnups are 25100 and 33200 MWd/MTM respectively. (44.196 cm (17.4") from the bottom of the rod).

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6. Conclusions

A study of the neutronics of the Saxton Plutonium Program-Irradiation in the Saxton Reactor was performed. It was confirmed during this analysis, that sufficient data exist to construct a HELIOS model.

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7. Acknowledgments

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9. Appendix

9.1. Group structures of the neutron transport calculations

Total number of energy groups - 190

Average lethargy and average energy per energy group:

1-6	2-5	3-4	4-2	5-1	6	7
2.000E+07eV	1.700E+07eV	1.492E+07eV	1.338E+07eV	1.200E+07eV	1.000E+07eV	8.825E+06eV
8	9	10	11	12	13	14
2.500E-01	3.000E-01	5.000E-01	6.500E-01	7.500E-01	8.000E-01	9.000E-01
7.788E+06eV	7.408E+06eV	6.065E+06eV	5.220E+06eV	4.724E+06eV	4.493E+06eV	4.066E+06eV
15	16	17	18	19	20	21
1.000E+00	1.150E+00	1.250E+00	1.300E+00	1.400E+00	1.442E+00	1.450E+00
3.679E+06eV	3.166E+06eV	2.865E+06eV	2.725E+06eV	2.466E+06eV	2.365E+06eV	2.346E+06eV
22	23	24	25	26	27	28
1.500E+00	1.600E+00	1.700E+00	1.750E+00	1.850E+00	2.000E+00	2.150E+00
2.231E+06eV	2.019E+06eV	1.827E+06eV	1.738E+06eV	1.572E+06eV	1.353E+06eV	1.165E+06eV
29	30	31	32	33	34	35
2.250E+00	2.300E+00	2.500E+00	2.650E+00	2.750E+00	2.800E+00	3.000E+00
1.054E+06eV	1.003E+06eV	8.208E+05eV	7.065E+05eV	6.393E+05eV	6.081E+05eV	4.979E+05eV
36	37	38	39	40	41	42
3.150E+00	3.250E+00	3.300E+00	3.500E+00	3.650E+00	3.750E+00	3.800E+00
4.285E+05eV	3.877E+05eV	3.688E+05eV	3.020E+05eV	2.599E+05eV	2.352E+05eV	2.237E+05eV
43	44	45	46	47	48	49
4.000E+00	4.200E+00	4.250E+00	4.350E+00	4.500E+00	4.750E+00	5.000E+00
1.832E+05eV	1.500E+05eV	1.426E+05eV	1.291E+05eV	1.111E+05eV	8.652E+04eV	6.738E+04eV
50	51	52	53	54	55	56
5.250E+00	5.500E+00	5.625E+00	5.750E+00	5.875E+00	5.950E+00	6.000E+00
5.247E+04eV	4.087E+04eV	3.607E+04eV	3.183E+04eV	2.809E+04eV	2.606E+04eV	2.479E+04eV
57	58	59	60	61	62	63
6.125E+00	6.250E+00	6.375E+00	6.500E+00	6.625E+00	6.750E+00	6.875E+00
2.187E+04eV	1.930E+04eV	1.704E+04eV	1.503E+04eV	1.327E+04eV	1.171E+04eV	1.033E+04eV
64	65	66	67	68	69	70
7.000E+00	7.125E+00	7.250E+00	7.375E+00	7.500E+00	7.625E+00	7.750E+00
9.119E+03eV	8.047E+03eV	7.102E+03eV	6.267E+03eV	5.531E+03eV	4.881E+03eV	4.307E+03eV
71	72	73	74	75	76	77
7.875E+00	8.000E+00	8.125E+00	8.250E+00	8.375E+00	8.500E+00	8.625E+00
3.801E+03eV	3.355E+03eV	2.960E+03eV	2.613E+03eV	2.306E+03eV	2.035E+03eV	1.796E+03eV
78	79	80	81	82	83	84
8.750E+00	8.875E+00	9.000E+00	9.125E+00	9.250E+00	9.375E+00	9.500E+00
1.585E+03eV	1.398E+03eV	1.234E+03eV	1.089E+03eV	9.611E+02eV	8.482E+02eV	7.485E+02eV
85	86	87	88	89	90	91
9.625E+00	9.750E+00	9.875E+00	1.000E+01	1.013E+01	1.025E+01	1.037E+01
6.606E+02eV	5.829E+02eV	5.144E+02eV	4.540E+02eV	4.006E+02eV	3.536E+02eV	3.120E+02eV
92	93	94	95	96	97	98
1.050E+01	1.062E+01	1.075E+01	1.087E+01	1.100E+01	1.113E+01	1.125E+01
2.754E+02eV	2.430E+02eV	2.144E+02eV	1.893E+02eV	1.670E+02eV	1.474E+02eV	1.301E+02eV
99	100	101	102	103	104	105
1.137E+01	1.150E+01	1.162E+01	1.175E+01	1.188E+01	1.200E+01	1.213E+01
1.148E+02eV	1.013E+02eV	8.940E+01eV	7.889E+01eV	6.962E+01eV	6.144E+01eV	5.422E+01eV
106	107	108	109	110	111	112
1.225E+01	1.237E+01	1.250E+01	1.262E+01	1.275E+01	1.287E+01	1.300E+01
4.785E+01eV	4.223E+01eV	3.727E+01eV	3.289E+01eV	2.902E+01eV	2.561E+01eV	2.260E+01eV
113	114	115	116	117	118	119
1.313E+01	1.325E+01	1.337E+01	1.350E+01	1.362E+01	1.375E+01	1.387E+01
1.995E+01eV	1.760E+01eV	1.554E+01eV	1.371E+01eV	1.210E+01eV	1.068E+01eV	9.422E+00eV
120	121	122	123	124	125	126
1.400E+01	1.413E+01	1.419E+01	1.425E+01	1.438E+01	1.450E+01	1.462E+01
8.315E+00eV	7.338E+00eV	6.868E+00eV	6.476E+00eV	5.715E+00eV	5.043E+00eV	4.451E+00eV
127	128	129	130	131	132	133
1.475E+01	1.488E+01	1.500E+01	1.512E+01	1.525E+01	1.538E+01	1.550E+01
3.928E+00eV	3.466E+00eV	3.059E+00eV	2.700E+00eV	2.382E+00eV	2.102E+00eV	1.855E+00eV
134	135	136	137	138	139	140
1.554E+01	1.557E+01	1.561E+01	1.565E+01	1.570E+01	1.574E+01	1.580E+01
1.790E+00eV	1.726E+00eV	1.659E+00eV	1.595E+00eV	1.525E+00eV	1.457E+00eV	1.381E+00eV
141	142	143	144	145	146	147
1.585E+01	1.591E+01	1.596E+01	1.600E+01	1.602E+01	1.605E+01	1.606E+01
1.308E+00eV	1.235E+00eV	1.166E+00eV	1.125E+00eV	1.099E+00eV	1.072E+00eV	1.062E+00eV
148	149	150	151	152	153	154
1.607E+01	1.608E+01	1.610E+01	1.613E+01	1.615E+01	1.617E+01	1.621E+01

	1.053E+00eV	1.043E+00eV	1.014E+00eV	9.920E-01eV	9.710E-01eV	9.506E-01eV	9.100E-01eV
155	1.625E+01	156 1.630E+01	157 1.636E+01	158 1.643E+01	159 1.652E+01	160 1.659E+01	161 1.668E+01
	8.764E-01eV	8.337E-01eV	7.821E-01eV	7.300E-01eV	6.700E-01eV	6.251E-01eV	5.700E-01eV
162	1.675E+01	163 1.680E+01	164 1.692E+01	165 1.699E+01	166 1.715E+01	167 1.726E+01	168 1.732E+01
	5.300E-01eV	5.032E-01eV	4.500E-01eV	4.170E-01eV	3.577E-01eV	3.206E-01eV	3.011E-01eV
169	1.735E+01	170 1.743E+01	171 1.750E+01	172 1.760E+01	173 1.781E+01	174 1.800E+01	175 1.804E+01
	2.907E-01eV	2.705E-01eV	2.510E-01eV	2.277E-01eV	1.844E-01eV	1.523E-01eV	1.457E-01eV

176	1.831E+01	177	1.862E+01	178	1.882E+01	179	1.898E+01	180	1.911E+01	181	1.927E+01	182	1.946E+01
	1.116E-01eV		8.197E-02eV		6.700E-02eV		5.692E-02eV		5.000E-02eV		4.275E-02eV		3.550E-02eV
183	1.960E+01	184	1.979E+01	185	2.001E+01	186	2.051E+01	187	2.118E+01	188	2.220E+01	189	2.330E+01
	3.061E-02eV		2.550E-02eV		2.049E-02eV		1.240E-02eV		6.325E-03eV		2.277E-03eV		7.602E-04eV
190	2.440E+01												
	2.540E-04eV												

9.2. Group structures of the gamma transport calculations

Total number of energy groups - 48

Energy group boundaries (eV):

5.000E+07	-1-	3.000E+07	-2-	2.000E+07	-3-	1.700E+07	-4-	1.400E+07	-5-	1.200E+07	-6-	1.000E+07
1.000E+07	-7-	9.000E+06	-8-	8.000E+06	-9-	7.500E+06	-10-	7.000E+06	-11-	6.500E+06	-12-	6.000E+06
6.000E+06	-13-	5.500E+06	-14-	5.000E+06	-15-	4.500E+06	-16-	4.000E+06	-17-	3.500E+06	-18-	3.000E+06
3.000E+06	-19-	2.666E+06	-20-	2.500E+06	-21-	2.333E+06	-22-	2.000E+06	-23-	1.875E+06	-24-	1.660E+06
1.660E+06	-25-	1.500E+06	-26-	1.330E+06	-27-	1.200E+06	-28-	1.125E+06	-29-	1.000E+06	-30-	9.000E+05
9.000E+05	-31-	8.000E+05	-32-	7.000E+05	-33-	6.000E+05	-34-	5.250E+05	-35-	5.000E+05	-36-	4.500E+05
4.500E+05	-37-	4.000E+05	-38-	3.000E+05	-39-	2.000E+05	-40-	1.500E+05	-41-	1.000E+05	-42-	8.000E+04
8.000E+04	-43-	6.000E+04	-44-	4.500E+04	-45-	3.000E+04	-46-	2.000E+04	-47-	1.000E+04	-48-	1.000E+03

9.3. Overall fission spectrum (HELIOS nuclear data library)

Total number of energy groups - 190

1	0.000006	2	0.000024	3	0.000071	4	0.000194	5	0.001084	6	0.001916	7	0.003854
8	0.002340	9	0.016040	10	0.021314	11	0.019609	12	0.011552	13	0.026700	14	0.031487
15	0.055696	16	0.042072	17	0.022238	18	0.046341	19	0.019936	20	0.003934	21	0.024154
22	0.048697	23	0.048402	24	0.023836	25	0.046490	26	0.065932	27	0.060366	28	0.036900
29	0.017403	30	0.062713	31	0.040034	32	0.023543	33	0.010895	34	0.038074	35	0.023232
36	0.013246	37	0.006038	38	0.020469	39	0.012128	40	0.006812	41	0.003065	42	0.010322
43	0.007805	44	0.001629	45	0.002927	46	0.003674	47	0.004594	48	0.003191	49	0.002202
50	0.001527	51	0.000576	52	0.000480	53	0.000402	54	0.000209	55	0.000127	56	0.000280
57	0.000235	58	0.000198	59	0.000165	60	0.000138	61	0.000115	62	0.000096	63	0.000076
64	0.000062	65	0.000052	66	0.000043	67	0.000036	68	0.000000	69	0.000000	70	0.000000
71	0.000000	72	0.000000	73	through 190 the same as above								

9.4. List of nuclides (HELIOS nuclear data library)

Total number of nuclides in the HELIOS nuclear data library: 271
 Number of elements in the HELIOS nuclear data library: 72

Elements in the HELIOS nuclear data library:

1. Effective nuclide	26. Holmium	51. Plutonium
2. Aluminum	27. Hydrogen	52. Rhodium
3. Americium	28. Indium	53. Ruthenium
4. Antimony	29. Iodine	54. Strontium
5. Barium	30. Iron	55. Silver
6. Boron	31. Krypton	56. Sulfur
7. Bromine	32. Lanthanum	57. Silicon
8. Beryllium	33. Lead	58. Sodium
9. Carbon	34. Lithium	59. Samarium
10. Cadmium	35. Lutetium	60. Tin
11. Calcium	36. Magnesium	61. Technetium
12. Cesium	37. Manganese	62. Tellurium
13. Cerium	38. Molybdenum	63. Tantalum
14. Chlorine	39. Neodymium	64. Titanium
15. Cobalt	40. Niobium	65. Terbium
16. Copper	41. Nitrogen	66. Thorium
17. Chromium	42. Nickel	67. Thulium
18. Curium	43. Neptunium	68. Uranium
19. Dysprosium	44. Oxygen	69. Vanadium
20. Erbium	45. Palladium	70. Xenon
21. Europium	46. Phosphorus	71. Yttrium
22. Fluorine	47. Promethium	72. Zirconium
23. Gadolinium	48. Praseodymium	
24. Gold	49. Potassium	
25. Hafnium	50. Protactinium	

The materials that are read from the HELIOS nuclear data library, which could be considered as an external component of the HERMES data base, are specified by their components (nuclides) and the number densities of these components. The nuclide identifiers are of the type 1,000Z+A, where Z is the atomic number and A is the mass in atomic mass units.

A = 000: identifies the element, and A = 1006, 1040, 6001, 8001, 26001, 28001, 40001, and 40002 and 501 < A < 800 have special meanings.

Available information in the HELIOS nuclear data library:

ONO.	ISOTOPE	TYPE	AT WEIGHT	ALFA	DELTA	RESONA	ABSORPTION	BURNABLE
GAMMA		(IDCODE)	(AMU)			TABLES	XS ONLY	ABSORBER
DATA								
1. Effective nuclide:								
1	1	0	1.000	-	-	NO	YES	NO
NO								
2. Molybdenum:								
2	42000	0	95.940	0.9592	0.0417	NO	NO	NO
YES								
3	42595	1	94.906	0.9587	0.0421	YES	NO	NO
YES								

4	42095	0	94.906	-	-	NO	YES	NO
YES								
5	42596	1	95.905	-	-	NO	YES	NO
YES								
6	42597	1	96.906	-	-	NO	YES	NO
YES								
7	42598	1	97.906	-	-	NO	YES	NO
YES								
8	42599	1	98.908	-	-	NO	YES	NO
YES								
9	42600	1	99.908	-	-	NO	YES	NO
YES								

3. Technetium:

10	43599	1	99.001	0.9604	0.0404	YES	NO	NO
YES								

4. Palladium:

11	46604	1	103.904	-	-	NO	YES	NO
YES								
12	46605	1	104.905	0.9626	0.0381	NO	NO	NO
YES								
13	46606	1	105.904	-	-	NO	YES	NO
YES								
14	46607	1	106.905	0.9633	0.0374	NO	NO	NO
YES								
15	46608	1	107.904	0.9636	0.0371	YES	NO	NO
YES								

5. Silver:

16	47107	0	106.905	0.9633	0.0374	YES	NO	NO
YES								
17	47109	0	108.905	0.9639	0.0367	YES	NO	NO
YES								
18	47609	1	108.905	0.9639	0.0367	NO	NO	NO
YES								
19	47710	1	109.906	-	-	NO	YES	NO
YES								
20	47611	1	110.906	-	-	NO	YES	NO
YES								

6. Indium:

21	49113	0	112.904	0.9652	0.0354	YES	NO	NO
YES								
22	49115	0	114.907	0.9658	0.0348	YES	NO	NO
YES								
23	49615	1	114.907	0.9658	0.0348	NO	NO	NO
YES								

7. Xenon:

24	54128	0	127.904	0.9692	0.0313	NO	NO	NO
YES								
25	54130	0	129.904	0.9697	0.0308	NO	NO	NO
YES								
26	54631	1	130.906	0.9699	0.0306	YES	NO	NO
YES								
27	54632	1	131.904	-	-	NO	YES	NO
YES								

28	54633	1	132.906	-	-	NO	YES	NO
YES								
29	54634	1	133.905	0.9706	0.0299	NO	NO	NO
YES								
30	54635	1	134.907	-	-	NO	YES	NO
YES								
31	54636	1	135.908	-	-	NO	YES	NO
YES								

8. Cesium:

32	55633	1	132.906	0.9704	0.0301	YES	NO	NO
YES								
33	55634	1	133.907	-	-	NO	YES	NO
YES								
34	55635	1	134.906	-	-	NO	YES	NO
YES								
35	55636	1	135.908	-	-	NO	YES	NO
YES								
36	55637	1	136.907	-	-	NO	YES	NO
YES								

9. Gadolinium:

37	64152	0	151.920	0.9740	0.0263	NO	NO	NO
YES								
38	64154	0	153.921	0.9743	0.0260	NO	NO	NO
YES								
39	64654	1	153.921	-	-	NO	YES	NO
YES								
40	64155	0	154.923	0.9745	0.0258	YES	NO	NO
YES								
41	64655	1	154.923	0.9745	0.0258	NO	NO	NO
YES								
42	64156	0	155.922	0.9747	0.0257	YES	NO	NO
YES								
43	64656	1	155.922	0.9747	0.0257	NO	NO	NO
YES								
44	64657	1	156.924	0.9748	0.0255	NO	NO	NO
YES								
45	64157	0	156.924	0.9748	0.0255	YES	NO	NO
YES								
46	64158	0	157.924	0.9750	0.0253	YES	NO	NO
YES								
47	64658	1	157.924	0.9750	0.0253	NO	NO	NO
YES								
48	64160	0	159.927	0.9753	0.0250	NO	NO	NO
YES								
49	64660	1	159.927	-	-	NO	YES	NO
YES								

10. Erbium:

50	68166	0	165.930	0.9762	0.0241	YES	NO	NO
YES								
51	68167	0	166.932	0.9763	0.0240	YES	NO	NO
YES								
52	68168	0	167.941	0.9765	0.0238	NO	NO	NO
YES								
53	68170	0	169.935	0.9767	0.0235	NO	NO	NO
YES								

11. Thulium:

54	69169	0	168.934	0.9766	0.0237	YES	NO	NO
YES								
55	69170	0	170.000	0.9767	0.0235	YES	NO	NO
YES								
56	69171	0	171.000	-	-	NO	YES	NO
YES								

12. Hafnium:

57	72174	0	173.940	0.9773	0.0230	NO	NO	NO
YES								
58	72176	0	175.941	0.9775	0.0227	YES	NO	NO
YES								
59	72177	0	176.943	0.9776	0.0226	YES	NO	NO
YES								
60	72178	0	177.944	0.9778	0.0225	YES	NO	NO
YES								
61	72179	0	178.946	0.9779	0.0224	YES	NO	NO
YES								
62	72180	0	179.947	0.9780	0.0222	YES	NO	NO
YES								

13. Thorium:

63	90230	3	230.036	0.9828	0.0174	NO	NO	NO
YES								
64	90232	2	232.033	0.9829	0.0172	YES	NO	NO
YES								
65	90233	3	233.042	0.9830	0.0172	NO	NO	NO
YES								
66	90234	3	234.044	0.9831	0.0171	NO	NO	NO
YES								

14. Uranium:

67	92232	3	232.033	0.9829	0.0172	NO	NO	NO
YES								
68	92233	3	233.045	0.9830	0.0172	YES	NO	NO
YES								
69	92234	3	234.041	0.9831	0.0171	NO	NO	NO
YES								
70	92235	3	235.044	0.9831	0.0170	YES	NO	NO
YES								
71	92236	3	236.046	0.9832	0.0169	YES	NO	NO
YES								
72	92237	3	237.048	0.9833	0.0169	NO	NO	NO
YES								
73	92238	2	238.051	0.9833	0.0168	YES	NO	NO
YES								

15. Plutonium:

74	94236	3	236.046	0.9832	0.0169	NO	NO	NO
YES								
75	94238	3	238.049	0.9833	0.0168	NO	NO	NO
YES								
76	94338	3	238.049	0.9833	0.0168	NO	NO	NO
YES								
77	94239	3	239.052	0.9834	0.0167	YES	NO	NO
YES								
78	94240	3	240.054	0.9835	0.0167	YES	NO	NO
YES								

79	94241	3	241.049	0.9835	0.0166	YES	NO	NO
YES								
80	94242	3	242.058	0.9836	0.0165	YES	NO	NO
YES								

16. Hydrogen:

81	1001	0	1.008	0.0000	11.0937	NO	NO	NO
YES								
82	1002	0	2.014	0.1132	2.1786	NO	NO	NO
YES								
83	1003	0	3.016	-	-	NO	YES	NO
YES								
84	1006	0	1.008	0.0000	11.0937	NO	NO	NO
YES								
85	1040	0	1.008	0.0000	11.0937	NO	NO	NO
YES								

17. Lithium:

86	3006	0	6.015	0.5111	0.6712	NO	NO	NO
YES								

18. Beryllium:

87	4009	0	9.012	0.6404	0.4457	NO	NO	NO
YES								

19. Boron:

88	5000	0	10.812	0.6900	0.3710	NO	NO	NO
YES								
89	5010	0	10.013	0.6698	0.4008	NO	NO	NO
YES								
90	5011	0	11.009	0.6947	0.3643	NO	NO	NO
YES								

20. Carbon:

91	6000	0	12.001	0.7160	0.3341	NO	NO	NO
YES								
92	6001	0	12.001	0.7160	0.3341	NO	NO	NO
YES								

21. Nitrogen:

93	7014	0	14.003	0.7512	0.2861	NO	NO	NO
YES								

22. Oxygen:

94	8001	0	15.991	0.7784	0.2505	NO	NO	NO
YES								

95	8016	0	15.991	0.7784	0.2505	NO	NO	NO
YES								

23. Fluorine:

96	9019	0	18.998	0.8100	0.2107	NO	NO	NO
YES								

24. Sodium:

97	11023	0	22.990	0.8402	0.1741	NO	NO	NO
YES								

25. Magnesium:

98	12000	0	24.305	0.8482	0.1647	NO	NO	NO
YES								

26. Aluminum:

99	13027	0	26.982	0.8622	0.1483	NO	NO	NO
YES								

27. Silicon:

100	14000	0	28.085	0.8672	0.1425	NO	NO	NO
YES								

28. Phosphorus:

101	15031	0	30.974	0.8788	0.1292	NO	NO	NO
YES								

29. Sulfur:

102	16000	0	32.064	0.8827	0.1248	NO	NO	NO
YES								

30. Chlorine:

103	17000	0	35.453	0.8933	0.1129	NO	NO	NO
YES								

31. Potassium:

104	19000	0	39.102	0.9027	0.1023	NO	NO	NO
YES								

32. Calcium:

105	20000	0	40.080	0.9050	0.0998	NO	NO	NO
YES								

33. Titanium:

106	22000	0	47.879	0.9198	0.0836	NO	NO	NO
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YES

34. Vanadium:

107	23000	0	50.942	0.9245	0.0785	NO	NO	NO
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YES

35. Chromium:

108	24000	0	51.996	0.9259	0.0769	NO	NO	NO
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YES

109	24050	0	49.946	0.9230	0.0801	NO	NO	NO
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YES

110	24052	0	51.940	0.9259	0.0770	NO	NO	NO
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YES

111	24053	0	52.941	0.9272	0.0756	NO	NO	NO
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YES

112	24054	0	53.939	0.9285	0.0742	NO	NO	NO
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YES

36. Manganese:

113	25055	0	54.938	0.9298	0.0728	NO	NO	NO
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YES

37. Iron:

114	26001	0	55.332	0.9303	0.0723	NO	NO	NO
YES								
115	26000	0	55.846	0.9309	0.0716	NO	NO	NO
YES								
116	26054	0	53.939	0.9285	0.0742	NO	NO	NO
YES								
117	26056	0	55.935	0.9310	0.0715	NO	NO	NO
YES								
118	26057	0	56.935	0.9321	0.0703	NO	NO	NO
YES								
119	26058	0	57.934	0.9333	0.0691	NO	NO	NO
YES								

38. Cobalt:

120	27059	0	58.933	0.9344	0.0679	NO	NO	NO
YES								

39. Nickel:

121	28001	0	58.722	0.9341	0.0681	NO	NO	NO
YES								
122	28000	0	58.688	0.9341	0.0682	NO	NO	NO
YES								
123	28058	0	57.936	0.9333	0.0690	NO	NO	NO
YES								
124	28059	0	59.118	0.9346	0.0677	NO	NO	NO
YES								
125	28060	0	59.931	0.9354	0.0667	NO	NO	NO
YES								
126	28061	0	60.931	0.9365	0.0657	NO	NO	NO
YES								
127	28062	0	61.928	0.9374	0.0646	NO	NO	NO
YES								
128	28064	0	63.928	0.9393	0.0626	NO	NO	NO
YES								

40. Copper:

129	29063	0	62.930	0.9384	0.0636	NO	NO	NO
YES								
130	29065	0	64.928	0.9402	0.0616	NO	NO	NO
YES								

41. Zirconium:

131	40090	0	89.904	0.9565	0.0445	NO	NO	NO
YES								
132	40591	1	90.903	-	-	NO	YES	NO
YES								
133	40000	0	91.220	0.9571	0.0439	NO	NO	NO
YES								
134	40001	0	91.220	0.9571	0.0439	NO	NO	NO
YES								
135	40002	0	91.511	0.9572	0.0437	NO	NO	NO
YES								
136	40593	1	92.906	-	-	NO	YES	NO
YES								
137	40595	1	94.908	-	-	NO	YES	NO
YES								
138	40596	1	95.905	-	-	NO	YES	NO
YES								

42. Niobium:

139	41093	0	92.903	0.9579	0.0431	NO	NO	NO
YES								
140	41595	1	94.907	-	-	NO	YES	NO
YES								

43. Ruthenium:

141	44600	1	99.904	-	-	NO	YES	NO
YES								
142	44601	1	100.906	0.9611	0.0396	NO	NO	NO
YES								
143	44602	1	101.904	-	-	NO	YES	NO
YES								
144	44603	1	102.906	-	-	NO	YES	NO
YES								
145	44604	1	103.906	-	-	NO	YES	NO
YES								
146	44605	1	104.908	-	-	NO	YES	NO
YES								
147	44606	1	105.908	-	-	NO	YES	NO
YES								

44. Rhodium:

148	45103	0	102.905	0.9619	0.0389	NO	NO	NO
YES								
149	45603	1	102.905	0.9619	0.0389	NO	NO	NO
YES								
150	45605	1	104.906	0.9626	0.0381	NO	NO	NO
YES								

45. Cadmium:

151	48000	0	112.426	0.9650	0.0356	NO	NO	NO
YES								
152	48110	0	109.903	0.9643	0.0364	NO	NO	NO
YES								
153	48610	1	109.903	-	-	NO	YES	NO
YES								
154	48111	0	110.905	0.9646	0.0361	NO	NO	NO
YES								
155	48611	1	110.905	-	-	NO	YES	NO
YES								
156	48112	0	111.903	0.9649	0.0357	NO	NO	NO
YES								
157	48113	0	112.900	0.9652	0.0354	NO	NO	NO
YES								
158	48613	1	112.900	-	-	NO	YES	NO
YES								
159	48114	0	113.903	0.9655	0.0351	NO	NO	NO
YES								

46. Tin:

160	50000	0	118.685	0.9669	0.0337	NO	NO	NO
YES								
161	50112	0	111.901	0.9649	0.0357	NO	NO	NO
YES								
162	50114	0	113.898	0.9655	0.0351	NO	NO	NO
YES								

163	50115	0	114.903	0.9658	0.0348	NO	NO	NO
YES								
164	50116	0	115.902	0.9661	0.0345	NO	NO	NO
YES								
165	50117	0	116.903	0.9664	0.0342	NO	NO	NO
YES								
166	50118	0	117.902	0.9666	0.0339	NO	NO	NO
YES								
167	50119	0	118.903	0.9669	0.0336	NO	NO	NO
YES								
168	50120	0	119.902	0.9672	0.0334	NO	NO	NO
YES								
169	50122	0	121.903	0.9677	0.0328	NO	NO	NO
YES								
170	50124	0	123.905	0.9682	0.0323	NO	NO	NO
YES								
171	50125	0	124.908	-	-	NO	YES	NO
YES								

47. Antimony:

172	51000	0	121.758	0.9677	0.0329	NO	NO	NO
YES								
173	51500	1	121.758	-	-	NO	YES	NO
YES								
174	51121	0	120.904	0.9675	0.0331	NO	NO	NO
YES								
175	51123	0	122.904	0.9680	0.0325	NO	NO	NO
YES								
176	51625	1	124.905	-	-	NO	YES	NO
YES								
177	51627	1	126.907	-	-	NO	YES	NO
YES								

48. Neodymium:

178	60642	1	141.908	-	-	NO	YES	NO
YES								
179	60643	1	142.910	0.9724	0.0280	NO	NO	NO
YES								
180	60644	1	143.910	-	-	NO	YES	NO
YES								
181	60645	1	144.913	0.9728	0.0276	NO	NO	NO
YES								
182	60646	1	145.913	-	-	NO	YES	NO
YES								
183	60647	1	146.916	-	-	NO	YES	NO
YES								
184	60648	1	147.917	-	-	NO	YES	NO
YES								
185	60650	1	149.921	-	-	NO	YES	NO
YES								

49. Promethium:

186	61647	1	146.915	0.9731	0.0272	NO	NO	NO
YES								
187	61748	1	147.918	0.9733	0.0270	NO	NO	NO
YES								
188	61648	1	147.918	-	-	NO	YES	NO
YES								
189	61649	1	148.918	-	-	NO	YES	NO
YES								

190	61651	1	150.921	-	-	NO	YES	NO
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YES

50. Samarium:

191	62647	1	146.915	0.9731	0.0272	NO	NO	NO
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YES

192	62648	1	147.915	-	-	NO	YES	NO
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YES

193	62649	1	148.917	0.9735	0.0269	NO	NO	NO
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YES

194	62650	1	149.917	0.9737	0.0267	NO	NO	NO
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YES

195	62651	1	150.919	0.9738	0.0265	NO	NO	NO
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YES

196	62652	1	151.920	0.9740	0.0263	NO	NO	NO
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YES

197	62653	1	152.922	-	-	NO	YES	NO
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YES

198	62654	1	153.922	-	-	NO	YES	NO
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YES

51. Europium:

199	63651	1	150.919	-	-	NO	YES	NO
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YES

200	63653	1	152.922	0.9742	0.0262	NO	NO	NO
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YES

201	63654	1	153.922	0.9743	0.0260	NO	NO	NO
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YES

202	63655	1	154.923	0.9745	0.0258	NO	NO	NO
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YES

203	63656	1	155.924	-	-	NO	YES	NO
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YES

204	63657	1	156.925	-	-	NO	YES	NO
-----	-------	---	---------	---	---	----	-----	----

YES

52. Lutetium:

205	71176	0	175.941	0.9775	0.0227	NO	NO	NO
YES								

53. Tantalum:

206	73181	0	180.954	0.9781	0.0221	NO	NO	NO
YES								
207	73182	0	181.950	0.9783	0.0220	NO	NO	NO
YES								

54. Gold:

208	79197	0	196.966	0.9799	0.0203	NO	NO	NO
YES								

55. Lead:

209	82206	0	205.969	0.9808	0.0194	NO	NO	NO
YES								
210	82207	0	206.978	0.9809	0.0193	NO	NO	NO
YES								
211	82208	0	207.977	0.9810	0.0192	NO	NO	NO
YES								

56. Protactinium:

212	91231	3	231.035	0.9828	0.0173	NO	NO	NO
YES								
213	91232	1	232.039	-	-	NO	YES	NO
YES								
214	91233	3	233.040	0.9830	0.0172	NO	NO	NO
YES								
215	91234	1	234.043	-	-	NO	YES	NO
YES								

57. Neptunium:

216	93237	3	237.048	0.9833	0.0169	NO	NO	NO
YES								
217	93238	3	238.051	0.9833	0.0168	NO	NO	NO
YES								
218	93239	3	239.053	0.9834	0.0167	NO	NO	NO
YES								

58. Americium:

219	95241	3	241.057	0.9835	0.0166	NO	NO	NO
YES								
220	95242	3	242.059	0.9836	0.0165	NO	NO	NO
YES								
221	95342	3	242.059	0.9836	0.0165	NO	NO	NO
YES								
222	95243	3	243.061	0.9837	0.0165	NO	NO	NO
YES								

59. Curium:

223	96242	3	242.058	0.9836	0.0165	NO	NO	NO
YES								
224	96243	3	243.061	0.9837	0.0165	NO	NO	NO
YES								

225	96244	3	244.063	0.9837	0.0164	NO	NO	NO
YES								
226	96245	3	245.065	0.9838	0.0163	NO	NO	NO
YES								
227	96246	3	246.067	0.9839	0.0163	NO	NO	NO
YES								

60. Bromine:

228	35581	1	80.916	-	-	NO	YES	NO
YES								

61. Krypton:

229	36583	1	82.914	-	-	NO	YES	NO
YES								
230	36585	1	84.912	-	-	NO	YES	NO
YES								

62. Strontium:

231	38589	1	88.908	-	-	NO	YES	NO
YES								
232	38590	1	89.908	-	-	NO	YES	NO
YES								

63. Yttrium:

233	39589	1	88.906	-	-	NO	YES	NO
YES								
234	39590	1	89.907	-	-	NO	YES	NO
YES								
235	39591	1	90.907	-	-	NO	YES	NO
YES								

64. Tellurium:

236	52727	1	126.905	-	-	NO	YES	NO
YES								
237	52729	1	128.907	-	-	NO	YES	NO
YES								
238	52632	1	131.908	-	-	NO	YES	NO
YES								

65. Iodine:

239	53627	1	126.904	-	-	NO	YES	NO
YES								
240	53629	1	128.905	-	-	NO	YES	NO
YES								
241	53631	1	130.907	-	-	NO	YES	NO
YES								
242	53635	1	134.910	-	-	NO	YES	NO
YES								

66. Barium:

243	56634	1	133.904	-	-	NO	YES	NO
YES								
244	56637	1	136.905	-	-	NO	YES	NO
YES								
245	56640	1	139.911	-	-	NO	YES	NO
YES								

67. Lanthanum:

246	57639	1	138.906	-	-	NO	YES	NO
YES								
247	57640	1	139.910	-	-	NO	YES	NO
YES								

68. Cerium:

248	58641	1	140.908	-	-	NO	YES	NO
YES								
249	58642	1	141.909	-	-	NO	YES	NO
YES								
250	58643	1	142.913	-	-	NO	YES	NO
YES								
251	58644	1	143.913	-	-	NO	YES	NO
YES								

69. Praseodymium:

252	59641	1	140.907	-	-	NO	YES	NO
YES								
253	59643	1	142.911	-	-	NO	YES	NO
YES								

70. Terbium:

254	65159	1	158.925	-	-	NO	YES	NO
YES								
255	65160	1	158.927	-	-	NO	YES	NO
YES								
256	65161	1	160.928	-	-	NO	YES	NO
YES								
257	65659	1	158.925	-	-	NO	YES	NO
YES								

258	65660	1	158.927	-	-	NO	YES	NO
YES								
259	65661	1	160.928	-	-	NO	YES	NO
YES								

71. Dysprosium:

260	66160	1	159.925	-	-	NO	YES	NO
YES								
261	66161	1	160.927	-	-	NO	YES	NO
YES								
262	66162	1	161.927	-	-	NO	YES	NO
YES								
263	66163	1	162.929	-	-	NO	YES	NO
YES								
264	66164	1	163.928	-	-	NO	YES	NO
YES								
265	66660	1	159.925	-	-	NO	YES	NO
YES								
266	66661	1	160.927	-	-	NO	YES	NO
YES								
267	66662	1	161.927	-	-	NO	YES	NO
YES								
268	66663	1	162.929	-	-	NO	YES	NO
YES								
269	66664	1	163.928	-	-	NO	YES	NO
YES								

72. Holmium:

270	67165	1	164.930	-	-	NO	YES	NO
YES								
271	67665	1	164.930	-	-	NO	YES	NO
YES								

9.5. List of nuclides with gamma data provided (HELIOS nuclear data library)

Total number of nuclides with gamma data provided in the HELIOS nuclear data library - 270

ONO.	ISOTOPE	AT NUMBER	TEXT
1	42595	42.00	Gamma data read from HERMES library
2	43599	43.00	Gamma data read from HERMES library
3	46608	46.00	Gamma data read from HERMES library
4	47107	47.00	Gamma data read from HERMES library
5	47109	47.00	Gamma data read from HERMES library
6	49113	49.00	Gamma data read from HERMES library
7	49115	49.00	Gamma data read from HERMES library
8	54631	54.00	Gamma data read from HERMES library
9	55633	55.00	Gamma data read from HERMES library
10	64155	64.00	Gamma data read from HERMES library
11	64156	64.00	Gamma data read from HERMES library
12	64157	64.00	Gamma data read from HERMES library
13	64158	64.00	Gamma data read from HERMES library
14	68166	68.00	Gamma data read from HERMES library
15	68167	68.00	Gamma data read from HERMES library
16	69169	69.00	Gamma data read from HERMES library
17	69170	69.00	Gamma data read from HERMES library
18	72176	72.00	Gamma data read from HERMES library
19	72177	72.00	Gamma data read from HERMES library
20	72178	72.00	Gamma data read from HERMES library
21	72179	72.00	Gamma data read from HERMES library
22	72180	72.00	Gamma data read from HERMES library
23	90232	90.00	Gamma data read from HERMES library
24	92233	92.00	Gamma data read from HERMES library
25	92235	92.00	Gamma data read from HERMES library
26	92236	92.00	Gamma data read from HERMES library
27	92238	92.00	Gamma data read from HERMES library
28	94239	94.00	Gamma data read from HERMES library
29	94240	94.00	Gamma data read from HERMES library
30	94241	94.00	Gamma data read from HERMES library
31	94242	94.00	Gamma data read from HERMES library
32	1001	1.00	Gamma data read from HERMES library
33	1002	1.00	Gamma data read from HERMES library
34	1006	1.00	Gamma data read from HERMES library
35	1040	1.00	Gamma data read from HERMES library
36	3006	3.00	Gamma data read from HERMES library
37	4009	4.00	Gamma data read from HERMES library
38	5000	5.00	Gamma data read from HERMES library
39	5010	5.00	Gamma data read from HERMES library
40	5011	5.00	Gamma data read from HERMES library
41	6000	6.00	Gamma data read from HERMES library
42	6001	6.00	Gamma data read from HERMES library
43	7014	7.00	Gamma data read from HERMES library
44	8001	8.00	Gamma data read from HERMES library
45	8016	8.00	Gamma data read from HERMES library
46	9019	9.00	Gamma data read from HERMES library
47	11023	11.00	Gamma data read from HERMES library
48	12000	12.00	Gamma data read from HERMES library
49	13027	13.00	Gamma data read from HERMES library
50	14000	14.00	Gamma data read from HERMES library
51	15031	15.00	Gamma data read from HERMES library
52	16000	16.00	Gamma data read from HERMES library
53	17000	17.00	Gamma data read from HERMES library
54	19000	19.00	Gamma data read from HERMES library
55	20000	20.00	Gamma data read from HERMES library
56	22000	22.00	Gamma data read from HERMES library
57	23000	23.00	Gamma data read from HERMES library
58	24000	24.00	Gamma data read from HERMES library

59	24050	24.00	Gamma data read from HERMES library
60	24052	24.00	Gamma data read from HERMES library
61	24053	24.00	Gamma data read from HERMES library
62	24054	24.00	Gamma data read from HERMES library
63	25055	25.00	Gamma data read from HERMES library
64	26001	26.00	Gamma data read from HERMES library
65	26000	26.00	Gamma data read from HERMES library
66	26054	26.00	Gamma data read from HERMES library
67	26056	26.00	Gamma data read from HERMES library
68	26057	26.00	Gamma data read from HERMES library
69	26058	26.00	Gamma data read from HERMES library
70	27059	27.00	Gamma data read from HERMES library
71	28001	28.00	Gamma data read from HERMES library
72	28000	28.00	Gamma data read from HERMES library
73	28058	28.00	Gamma data read from HERMES library
74	28059	28.00	Gamma data read from HERMES library
75	28060	28.00	Gamma data read from HERMES library
76	28061	28.00	Gamma data read from HERMES library
77	28062	28.00	Gamma data read from HERMES library
78	28064	28.00	Gamma data read from HERMES library
79	29063	29.00	Gamma data read from HERMES library
80	29065	29.00	Gamma data read from HERMES library
81	40000	40.00	Gamma data read from HERMES library
82	40001	40.00	Gamma data read from HERMES library
83	40002	40.00	Gamma data read from HERMES library
84	40090	40.00	Gamma data read from HERMES library
85	41093	41.00	Gamma data read from HERMES library
86	42000	42.00	Gamma data read from HERMES library
87	44601	44.00	Gamma data read from HERMES library
88	45103	45.00	Gamma data read from HERMES library
89	45603	45.00	Gamma data read from HERMES library
90	45605	45.00	Gamma data read from HERMES library
91	46605	46.00	Gamma data read from HERMES library
92	46607	46.00	Gamma data read from HERMES library
93	47609	47.00	Gamma data read from HERMES library
94	48000	48.00	Gamma data read from HERMES library
95	48110	48.00	Gamma data read from HERMES library
96	48111	48.00	Gamma data read from HERMES library
97	48112	48.00	Gamma data read from HERMES library
98	48113	48.00	Gamma data read from HERMES library
99	48114	48.00	Gamma data read from HERMES library
100	49615	49.00	Gamma data read from HERMES library
101	50000	50.00	Gamma data read from HERMES library
102	50112	50.00	Gamma data read from HERMES library
103	50114	50.00	Gamma data read from HERMES library
104	50115	50.00	Gamma data read from HERMES library
105	50116	50.00	Gamma data read from HERMES library
106	50117	50.00	Gamma data read from HERMES library
107	50118	50.00	Gamma data read from HERMES library
108	50119	50.00	Gamma data read from HERMES library
109	50120	50.00	Gamma data read from HERMES library
110	50122	50.00	Gamma data read from HERMES library
111	50124	50.00	Gamma data read from HERMES library
112	51000	51.00	Gamma data read from HERMES library
113	51121	51.00	Gamma data read from HERMES library
114	51123	51.00	Gamma data read from HERMES library
115	54128	54.00	Gamma data read from HERMES library
116	54130	54.00	Gamma data read from HERMES library
117	54634	54.00	Gamma data read from HERMES library
118	60643	60.00	Gamma data read from HERMES library
119	60645	60.00	Gamma data read from HERMES library
120	61647	61.00	Gamma data read from HERMES library
121	61748	61.00	Gamma data read from HERMES library
122	62647	62.00	Gamma data read from HERMES library

123	62649	62.00	Gamma data read from HERMES library
124	62650	62.00	Gamma data read from HERMES library
125	62651	62.00	Gamma data read from HERMES library
126	62652	62.00	Gamma data read from HERMES library
127	63653	63.00	Gamma data read from HERMES library
128	63654	63.00	Gamma data read from HERMES library
129	63655	63.00	Gamma data read from HERMES library
130	64152	64.00	Gamma data read from HERMES library
131	64154	64.00	Gamma data read from HERMES library
132	64160	64.00	Gamma data read from HERMES library
133	64655	64.00	Gamma data read from HERMES library
134	64656	64.00	Gamma data read from HERMES library
135	64657	64.00	Gamma data read from HERMES library
136	64658	64.00	Gamma data read from HERMES library
137	71176	71.00	Gamma data read from HERMES library
138	68168	68.00	Gamma data read from HERMES library
139	68170	68.00	Gamma data read from HERMES library
140	72174	72.00	Gamma data read from HERMES library
141	73181	73.00	Gamma data read from HERMES library
142	73182	73.00	Gamma data read from HERMES library
143	79197	79.00	Gamma data read from HERMES library
144	82206	82.00	Gamma data read from HERMES library
145	82207	82.00	Gamma data read from HERMES library
146	82208	82.00	Gamma data read from HERMES library
147	90230	90.00	Gamma data read from HERMES library
148	90233	90.00	Gamma data read from HERMES library
149	90234	90.00	Gamma data read from HERMES library
150	91231	91.00	Gamma data read from HERMES library
151	91233	91.00	Gamma data read from HERMES library
152	92232	92.00	Gamma data read from HERMES library
153	92234	92.00	Gamma data read from HERMES library
154	92237	92.00	Gamma data read from HERMES library
155	93237	93.00	Gamma data read from HERMES library
156	93238	93.00	Gamma data read from HERMES library
157	93239	93.00	Gamma data read from HERMES library
158	94236	94.00	Gamma data read from HERMES library
159	94238	94.00	Gamma data read from HERMES library
160	94338	94.00	Gamma data read from HERMES library
161	95241	95.00	Gamma data read from HERMES library
162	95242	95.00	Gamma data read from HERMES library
163	95342	95.00	Gamma data read from HERMES library
164	95243	95.00	Gamma data read from HERMES library
165	96242	96.00	Gamma data read from HERMES library
166	96243	96.00	Gamma data read from HERMES library
167	96244	96.00	Gamma data read from HERMES library
168	96245	96.00	Gamma data read from HERMES library
169	96246	96.00	Gamma data read from HERMES library
170	1003	1.00	Gamma data read from HERMES library
171	35581	35.00	Gamma data read from HERMES library
172	36583	36.00	Gamma data read from HERMES library
173	36585	36.00	Gamma data read from HERMES library
174	38589	38.00	Gamma data read from HERMES library
175	38590	38.00	Gamma data read from HERMES library
176	39589	39.00	Gamma data read from HERMES library
177	39590	39.00	Gamma data read from HERMES library
178	39591	39.00	Gamma data read from HERMES library
179	40591	40.00	Gamma data read from HERMES library
180	40593	40.00	Gamma data read from HERMES library
181	40595	40.00	Gamma data read from HERMES library
182	40596	40.00	Gamma data read from HERMES library
183	41595	41.00	Gamma data read from HERMES library
184	42095	42.00	Gamma data read from HERMES library
185	42596	42.00	Gamma data read from HERMES library
186	42597	42.00	Gamma data read from HERMES library

187	42598	42.00	Gamma data read from HERMES library
188	42599	42.00	Gamma data read from HERMES library
189	42600	42.00	Gamma data read from HERMES library
190	44600	44.00	Gamma data read from HERMES library
191	44602	44.00	Gamma data read from HERMES library
192	44603	44.00	Gamma data read from HERMES library
193	44604	44.00	Gamma data read from HERMES library
194	44605	44.00	Gamma data read from HERMES library
195	44606	44.00	Gamma data read from HERMES library
196	46604	46.00	Gamma data read from HERMES library
197	46606	46.00	Gamma data read from HERMES library
198	47710	47.00	Gamma data read from HERMES library
199	47611	47.00	Gamma data read from HERMES library
200	48610	48.00	Gamma data read from HERMES library
201	48611	48.00	Gamma data read from HERMES library
202	48613	48.00	Gamma data read from HERMES library
203	50125	50.00	Gamma data read from HERMES library
204	51500	51.00	Gamma data read from HERMES library
205	51625	51.00	Gamma data read from HERMES library
206	51627	51.00	Gamma data read from HERMES library
207	52727	52.00	Gamma data read from HERMES library
208	52729	52.00	Gamma data read from HERMES library
209	52632	52.00	Gamma data read from HERMES library
210	53627	53.00	Gamma data read from HERMES library
211	53629	53.00	Gamma data read from HERMES library
212	53631	53.00	Gamma data read from HERMES library
213	53635	53.00	Gamma data read from HERMES library
214	54632	54.00	Gamma data read from HERMES library
215	54633	54.00	Gamma data read from HERMES library
216	54635	54.00	Gamma data read from HERMES library
217	54636	54.00	Gamma data read from HERMES library
218	55634	55.00	Gamma data read from HERMES library
219	55635	55.00	Gamma data read from HERMES library
220	55636	55.00	Gamma data read from HERMES library
221	55637	55.00	Gamma data read from HERMES library
222	56634	56.00	Gamma data read from HERMES library
223	56637	56.00	Gamma data read from HERMES library
224	56640	56.00	Gamma data read from HERMES library
225	57639	57.00	Gamma data read from HERMES library
226	57640	57.00	Gamma data read from HERMES library
227	58641	58.00	Gamma data read from HERMES library
228	58642	58.00	Gamma data read from HERMES library
229	58643	58.00	Gamma data read from HERMES library
230	58644	58.00	Gamma data read from HERMES library
231	59641	59.00	Gamma data read from HERMES library
232	59643	59.00	Gamma data read from HERMES library
233	60642	60.00	Gamma data read from HERMES library
234	60644	60.00	Gamma data read from HERMES library
235	60646	60.00	Gamma data read from HERMES library
236	60647	60.00	Gamma data read from HERMES library
237	60648	60.00	Gamma data read from HERMES library
238	60650	60.00	Gamma data read from HERMES library
239	61648	61.00	Gamma data read from HERMES library
240	61649	61.00	Gamma data read from HERMES library
241	61651	61.00	Gamma data read from HERMES library
242	62648	62.00	Gamma data read from HERMES library
243	62653	62.00	Gamma data read from HERMES library
244	62654	62.00	Gamma data read from HERMES library
245	63651	63.00	Gamma data read from HERMES library
246	63656	63.00	Gamma data read from HERMES library
247	63657	63.00	Gamma data read from HERMES library
248	64654	64.00	Gamma data read from HERMES library
249	64660	64.00	Gamma data read from HERMES library
250	65159	65.00	Gamma data read from HERMES library

251	65160	65.00	Gamma data read from HERMES library
252	65161	65.00	Gamma data read from HERMES library
253	65659	65.00	Gamma data read from HERMES library
254	65660	65.00	Gamma data read from HERMES library
255	65661	65.00	Gamma data read from HERMES library
256	66160	66.00	Gamma data read from HERMES library
257	66161	66.00	Gamma data read from HERMES library
258	66162	66.00	Gamma data read from HERMES library
259	66163	66.00	Gamma data read from HERMES library
260	66164	66.00	Gamma data read from HERMES library
261	66660	66.00	Gamma data read from HERMES library
262	66661	66.00	Gamma data read from HERMES library
263	66662	66.00	Gamma data read from HERMES library
264	66663	66.00	Gamma data read from HERMES library
265	66664	66.00	Gamma data read from HERMES library
266	67165	67.00	Gamma data read from HERMES library
267	67665	67.00	Gamma data read from HERMES library
268	69171	69.00	Gamma data read from HERMES library
269	91232	91.00	Gamma data read from HERMES library
270	91234	91.00	Gamma data read from HERMES library

9.6. Description of the HELIOS Input file for the Unified Saxton Reactor Core Model

Within the framework of the recent study the unified model has been created in order to maintain capability to analyze characteristics of the Saxton Partial Plutonium Core (SPPC) while taking into account all system complexities. A Theoretical model called Unified Saxton Reactor Core Model (USRC) was described above in the main report sections.

Original HELIOS input was organized as set of four input files with the following contents:

- Material compositions
- Assemblies differential information
- Reactor core geometry
- Burnup and output flow

Some sections of the HELIOS input file components are presented below:

Material compositions:

```
+SET management.

&ADD      = SET ( 'USRC01.hrf' / SAXTON; AURORA; &v305A )

! Material compositions: Fuel compositions:      UOX, MOXP, MOXV;
!                               Non-fuel compositions: SS304, ZR40, INCONEL, AL, MCR, H2O !
! ===== !

'UOX1'    = MAT ( 10.1800/ 92235, 5.059582;
                92238, 83.08645;
                8001, 11.85397)                                !Fuel material !

'MOXP'    = MAT ( 10.3334/ 94239, 5.2649796;
                94240, 0.4986283;
                94241, 0.0517829;
                94242, 2.32732E-03;
                92235, 0.5895384;
                92238, 81.7482208;
                8001, 11.8439409)                                !Fuel material !

'MOXV'    = MAT ( 9.5639/ 94239, 5.2649796;
                94240, 0.4986283;
```

```
94241, 0.0517829;  
94242, 2.32732E-03;  
92235, 0.5895384;  
92238, 81.7482208;  
8001, 11.8439409)
```

!Fuel material !

```
'SS304' = MAT ( 7.9400/  
26054, 3.89670;  
26056, 62.8161;  
26057, 1.46390;  
26058, 0.19830;  
24050, 0.79310;  
24052, 15.9030;  
24053, 1.83790;  
24054, 0.46610;  
28058, 6.40250;  
28060, 2.53200;  
28061, 0.11140;  
28062, 0.35990;  
28064, 0.09410;  
25055, 2.00000;  
14000, 1.00000;  
6000, 0.08000;  
15031, 0.04570 )
```

! Clad, grid material !

```

'ZR40'      = MAT ( 6.5800/ 40000, 98.2000;
                50112, 0.01370;
                50114, 0.00940;
                50115, 0.00520;
                50116, 0.21280;
                50117, 0.11340;
                50118, 0.36080;
                50119, 0.12890;
                50120, 0.49380;
                50122, 0.07130;
                50124, 0.09060;
                26054, 0.01140;
                26056, 0.18370;
                26057, 0.00430;
                26058, 0.00060;
                24050, 0.00420;
                24052, 0.08370;
                24053, 0.00970;
                24054, 0.000001 )
                                                    ! Clad material !

'INCONEL' = MAT ( 8.3000/ 26054, 0.40000;
                26056, 6.43000;
                26057, 0.15000;
                26058, 0.02000;
                24050, 0.63000;
                24052, 12.5600;
                24053, 1.45000;
                24054, 0.37000;
                28058, 49.2000;
                28060, 19.4600;
                28061, 0.86000;
                28062, 2.77000;
                28064, 0.72000;
                14000, 2.50000;
                22000, 2.50000 )
                                                    ! Grid material !

'AL'       = MAT ( 2.7020/ 13027, 100.000 )
                                                    ! Grid material !

'MCR'     = MAT ( 9.9271/ 47107, 41.1000;
                47109, 38.9000;
                49113, 0.63400;
                49115, 14.4000;
                48000, 5.00000 )
                                                    ! Control rod !

'B'       = MAT ( 1.8 / 5000, 100.000 )
                                                    ! B !

'H2O'    = MAT ( 0.7656/ 1001, 66.6700;
                8016, 33.3300 )
                                                    ! Coolant !

&ADD     = SET ( 'USRC01.hrf' / SAXTON; AURORA; &v305A )
&MAT     = SET ( 'USRC01.hrf' / SAXTON; AURORA )

```

```

!
! Model
!

```

Geometry

```

$p      = PAR ( "1.4732" )
$rfuel  = PAR("$FuelDiameter/2")
$rho    = PAR("$HoleDiameter/2")
$rcld   = PAR("$CladDiameter/2")
$rtube  = PAR("$TubeDiameter/2")

$rw     = PAR("$rtube-$TubeThickness")

```

```

Fpin    = CCS($rhole,"$rhole+0.9*$thfu", $rfu,$rclad // hole,fuel,fuel,clad)
Bpin    = CCS($rhole,"$rhole+0.3*$thfu","$rhole+0.5*$thfu","$rhole+0.7*$thfu",
          "$rhole+0.9*$thfu", $rfu,$rclad
          // hole,fuel,fuel,fuel,fuel,fuel,clad)

Gpin    = CCS("0.4*$rw","0.7*$rw",$rw,$rtube /6(2,4)/
          tube,tube,tube,tube,tube,tube,tube,
          tube,tube,tube,tube,tube,tube,
          clad,clad,clad,clad,clad,clad)

pin1    = CCS ( 0.44, $rp1 // fuel, clad )
pin2    = CCS ( 0.50, $rp2 // fuel, clad )
pin3    = CCS ( 0.55, $rp3 // water, clad )
pin4    = CCS ( .20, .30, .35, .40, .45, .50, $rp4 //
          fuel,fuel,fuel,fuel,fuel,fuel, clad )

'1.8'   = STR ( $cell1 )
'2.0'   = STR ( $cell1 )
'2.4'   = STR ( $cell2 )
'2.6'   = STR ( $cell2 )
'3.0'   = STR ( $cell2 )
WP      = STR ( $cell3 )
BA      = STR ( $cell4 )

FPin    = CCS($rhole,$rfu,$rclad // hole,fuel,clad)
BPin    = CCS($rhole,"$rhole+0.3*$thfu","$rhole+0.6*$thfu","$rhole+0.8*$thfu",
          $rfu,$rclad // hole,fuel,fuel,fuel,fuel,clad)
GPin    = CCS("0.4*$rw","0.7*$rw",$rw,$rtube //tube,tube,tube,clad)

$D      = PAR("$rtube*3**0.5/2")
$E      = PAR("$rtube/2")
$F      = PAR("$rtube")

$c1     = PAR ( "$delw"           )           !Centre piece: 1st mesh line!
$c2     = PAR ( "2*$delw"        )           !Centre piece: 2nd mesh line!
$c3     = PAR ( "3*$delw"        )           !Centre piece: 3rd mesh line!
$c4     = PAR ( "3*$delw+$w1"    )           !Centre piece: 4th mesh line!

wg      = STR ( (0,0)           (0,$w4)       ($p,$w4)       ($p,0)           ! 1- 4 !
               (" $p/2",0)           ! 5 !
               (0,$film) (" $p/2", $film) ($p,$film)       ! 6- 8 !
               (0,$w1) (" $p/2", $w1) ($p,$w1)           ! 9-11 !
               (0,$w2) (" $p/2", $w2) ($p,$w2)           ! 12-14 !
               (0,$w3) (" $p/2", $w3) ($p,$w3)           ! 15-17 !
               (" $p/2", $w4)           ! 18 !
               / 4,gap / /           1, 6, 7, 5,film; 5, 7, 8, 4,film;
               6, 9,10, 7,box; 7,10,11, 8,box; 9,12,13,10,gap;
               10,13,14,11,gap; 12,15,16,13,gap; 13,16,17,14,gap;
               15, 2,18,16,gap )

$Fcorn  = PAR((0,"-$c") ($x2,$y2) ($x3,$h3) ($a,$h3) ($a,"-$b")           ! 1-5 !
              ("-$a","-$b") ($x7,$y7) (0,$h1) ($a,$b)           ! 6-9 !
              ( $x10, $y10) ($x11,$h1) ($a,$h1)           ! 10-12 !
              ( $x13, $y13) ($x14,$h2) ($a,$h2)           ! 13-15 !
              ( 0,"-$f")("$-d","-$e")("$-d", $e )           ! 16-18 !
              ( 0, $f )($ d , $e )($ d , "$-e")           ! 19-20 !
              / 5,cool / Fpin(0,0)/ 1,6,17,16,cool; 6,7,18,17,cool; ! 1-3 !
              7,8,19,18,cool; 8,9,20,19,cool; 9,5,21,20,cool; ! 4-6 !
              5,1,16,21,cool; 6,10, 7,cool; 8,12, 9,cool; ! 7-9 !
              10,13,14,11,shroud; 11,14,15,12,shroud; ! 10-11 !
              13, 2, 3,14,gap; 14, 3, 4,15,gap ) ! 12-13 !

$Bcorn  = PAR((0,"-$c") ($x2,$y2) ($x3,$h3) ($a,$h3) ($a,"-$b")           ! 1-5 !

```

```

      ("-$a", "-$b") ($x7,$y7) (0,$h1) ($a,$b) ! 6-9 !
      ( $x10, $y10) ($x11,$h1) ($a,$h1) ! 10-12 !
      ( $x13, $y13) ($x14,$h2) ($a,$h2) ! 13-15 !
      ( 0,"-$f")("-$d", "-$e")("-$d", $e ) ! 16-18 !
      ( 0, $f )( $d , $e )( $d , "-$e") ! 19-20 !
      / 5,cool / Bpin(0,0) / 1,6,17,16,cool; 6,7,18,17,cool; ! 1-3 !
          7,8,19,18,cool; 8,9,20,19,cool; 9,5,21,20,cool; ! 4-6 !
          5,1,16,21,cool; 6,10, 7,cool; 8,12, 9,cool; ! 7-9 !
          10,13,14,11,shroud; 11,14,15,12,shroud; ! 10-11 !
          13, 2, 3,14,gap; 14, 3, 4,15,gap ) ! 12-13 !
wgx    = STR ( (0,0) (0,$w2) ($p,$w2) ($p,0) ! 1- 4!
              (" $p/2",0) ! 5 !
              (0,$film) (" $p/2", $film) ($p,$film) ! 6- 8!
              (0,$w1) (" $p/2", $w1) ($p,$w1) ! 9-11!
              (" $p/2", $w2) ! 12 !
              / 4,gap / / 1, 6, 7, 5,film; 5, 7, 8, 4,film;
                6, 9,10, 7,box; 7,10,11, 8,box; 9, 2,12,10,gap )

ng     = STR ( (0,0) (0,$n3) ($p,$n3) ($p,0) ! 1 - 4!
              (" $p/2",0) ! 5 !
              (0,$film) (" $p/2", $film) ($p,$film) ! 6 - 8!
              (0,$n1) (" $p/2", $n1) ($p,$n1) ! 9 - 11!
              (0,$n2) (" $p/2", $n2) ($p,$n2) ! 12-14!
              (" $p/2", $n3) !node 15 !
              / 4,gap / / 1, 6, 7, 5,film; 5, 7, 8, 4,film;
                6, 9,10, 7,box; 7,10,11, 8,box; 9,12,13,10,gap;
                10,13,14,11,gap; 12, 2,15,13,gap )

ww     = STR ( (0,0) (0,$w2) ($w2,$w2) ($w2,0) ! 1- 4!
              ($film,0) ($w1,0) (0,$film) ($film,$film) ! 5- 8!
              (0,$w1) ($w1,$w1) ($w2,$w1) ($w1,$w2) ! 9-12!
              / 4,gap / / 1, 9,10, 6,box; 1, 7, 8, 5,film;
                6,10,11, 4,gap; 9, 2,12,10,gap )

$Fcell = PAR(("-$a", "-$b")("-$a", $b )(0, $c ) ! 1-3 !
             ( $a , $b )( $a , "-$b")(0, "-$c") ! 4-6 !
             ("-$d", "-$e")("-$d", $e )(0, $f ) ! 7-9 !
             ( $d , $e )( $d , "-$e")(0, "-$f") ! 10-12 !
             / 6,cool / Fpin(0,0) / 1,2, 8, 7,cool; 2,3, 9, 8,cool; ! 1-3 !
                 3,4,10,9,cool; 4,5,11,10,cool; 5,6,12,11,cool) ! 4-6 !

$Bcell = PAR(("-$a", "-$b")("-$a", $b )(0, $c ) ! 1-3 !
             ( $a , $b )( $a , "-$b")(0, "-$c") ! 4-6 !
             ("-$d", "-$e")("-$d", $e )(0, $f ) ! 7-9 !
             ( $d , $e )( $d , "-$e")(0, "-$f") ! 10-12 !
             / 6,cool / Bpin(0,0) / 1,2, 8, 7,cool; 2,3, 9, 8,cool; ! 1-3 !
                 3,4,10,9,cool; 4,5,11,10,cool; 5,6,12,11,cool) ! 4-6 !

$Gcell = PAR(("-$a", "-$b")("-$a", $b )(0, $c ) ! 1-3 !
             ( $a , $b )( $a , "-$b")(0, "-$c") ! 4-6 !
             ("-$D", "-$E")("-$D", $E )(0, $F ) ! 7-9 !
             ( $D , $E )( $D , "-$E")(0, "-$F") ! 10-12 !
             / 6,cool / Gpin(0,0) / 1,2, 8, 7,cool; 2,3, 9, 8,cool; ! 1-3 !
                 3,4,10,9,cool; 4,5,11,10,cool; 5,6,12,11,cool) ! 4-6 !

$FCorn = PAR((0, "-$c") ($x2,$y2) ($x3,$h3) ($a,$h3) ($a, "-$b") ! 1-5 !
             ("-$a", "-$b") ($x7,$y7) (0,$h1) ($a,$b) ! 6-9 !
             ( $x10, $y10) ($x11,$h1) ($a,$h1) ! 10-12 !
             ( $x13, $y13) ($x14,$h2) ($a,$h2) ! 13-15 !
             / 5,cool / FPin(0,0) / 13,14,15,12,11,10,shroud; ! 1-2 !
                 2, 3, 4,15,14,13,gap; ! 3-3 !
                 6,10,7,cool; 7,11,8,cool; 8,12,9,cool) ! 4-6 !

$BCorn = PAR((0, "-$c") ($x2,$y2) ($x3,$h3) ($a,$h3) ($a, "-$b") ! 1-5 !

```

```

        ("-$a", "-$b") ($x7,$y7) (0,$h1) ($a,$b) ! 6-9 !
        ($x10, $y10) ($x11,$h1) ($a,$h1) ! 10-12 !
        ($x13, $y13) ($x14,$h2) ($a,$h2) ! 13-15 !
        / 5,cool / BPin(0,0) / 13,14,15,12,11,10,shroud; ! 1-2 !
            2, 3, 4,15,14,13,gap; ! 3-3 !
            6,10,7,cool; 7,11,8,cool; 8,12,9,cool) ! 4-6 !

$FCell = PAR(("-$a", "-$b")("-$a", $b )(0, $c ) ! 1-3 !
        ( $a , $b )( $a , "-$b")(0, "-$c") ! 4-6 !
        / 6,cool / FPin(0,0) / )

$BCell = PAR(("-$a", "-$b")("-$a", $b )(0, $c ) ! 1-3 !
        ( $a , $b )( $a , "-$b")(0, "-$c") ! 4-6 !
        / 6,cool / BPin(0,0) / )

$GCell = PAR(("-$a", "-$b")("-$a", $b )(0, $c ) ! 1-3 !
        ( $a , $b )( $a , "-$b")(0, "-$c") ! 4-6 !
        / 6,cool / GPin(0,0) / )

wn = STR ( (0,0) (0,$w4) ($n3,$w4) ($n3,0) ! 1- 4!
        ($film,0) ($n1,0) ($n2,0) ! 5- 7!
        (0,$film) ($film,$film) ! 8- 9!
        (0,$w1) ($n1,$w1) ($n2,$w1) ($n3,$w1) ! 10-13!
        (0,$w2) ($n1,$w2) ($n2,$w2) ($n3,$w2) ! 14-17!
        (0,$w3) ($n1,$w3) ($n2,$w3) ($n3,$w3) ! 18-21!
            ($n1,$w4) ($n2,$w4) ! 22-23!
        / 4,gap / / 1,10,11, 6,box; 1, 8, 9, 5,film;
        6,11,12, 7,gap; 7,12,13, 4,gap; 10,14,15,11,gap;
        11,15,16,12,gap; 12,16,17,13,gap; 14,18,19,15,gap;
        15,19,20,16,gap; 16,20,21,17,gap; 18, 2,22,19,gap;
        19,22,23,20,gap )

nw = STR ( (0,0) (0,$n3) ($w4,$n3) ($w4,0) ! 1- 4!
        ($film,0) ($w1,0) ($w2,0) ($w3,0) ! 5- 8!
        (0,$film) ($film,$film) ! 9-10!
        (0,$n1) ($w1,$n1) ($w2,$n1) ($w3,$n1) ($w4,$n1) ! 11-15!
        (0,$n2) ($w1,$n2) ($w2,$n2) ($w3,$n2) ($w4,$n2) ! 16-20!
            ($w1,$n3) ($w2,$n3) ($w3,$n3) ! 21-23!
        / 4,gap / / 1,11,12, 6,box; 1, 9,10, 5,film;
        6,12,13, 7,gap; 7,13,14, 8,gap; 8,14,15, 4,gap;
        11,16,17,12,gap; 12,17,18,13,gap; 13,18,19,14,gap;
        14,19,20,15,gap; 16, 2,21,17,gap; 17,21,22,18,gap;
        18,22,23,19,gap )

nn = STR ( (0,0) (0,$n3) ($n3,$n3) ($n3,0) ! 1- 4!
        ($film,0) ($n1,0) ($n2,0) ! 5- 7!
        (0,$film) ($film,$film) ! 8- 9!
        (0,$n1) ($n1,$n1) ($n2,$n1) ($n3,$n1) ! 10-13!
        (0,$n2) ($n1,$n2) ($n3,$n2) ! 14-16!
            ($n1,$n3) ($n2,$n3) ! 17-18!
        / 4,gap / dpin ($n3,$n3) 2 / 1,10,11, 6,box;
        1, 8, 9, 5,film; 6,11,12, 7,gap; 7,12,13, 4,gap;
        10,14,15,11,gap; 14, 2,17,15,gap )

$BCorn = PAR((0, "-$c") ($x2,$y2) ($x3,$h3) ($a,$h3) ($a, "-$b") ! 1-5 !
        ("-$a", "-$b") ($x7,$y7) (0,$h1) ($a,$b) ! 6-9 !
        ($x10, $y10) ($x11,$h1) ($a,$h1) ! 10-12 !
        ($x13, $y13) ($x14,$h2) ($a,$h2) ! 13-15 !
        / 5,cool / BPin(0,0) / 13,14,15,12,11,10,shroud; ! 1-2 !
            2, 3, 4,15,14,13,gap; ! 3-3 !
            6,10,7,cool; 7,11,8,cool; 8,12,9,cool) ! 4-6 !

$FCell = PAR(("-$a", "-$b")("-$a", $b )(0, $c ) ! 1-3 !
        ( $a , $b )( $a , "-$b")(0, "-$c") ! 4-6 !
        / 6,cool / FPin(0,0) / )

```

```

$BCell = PAR(("-$a", "-$b")("-$a", $b)(0, $c) ! 1-3 !
          ( $a , $b)( $a , "-$b")(0, "-$c") ! 4-6 !
          / 6,cool / BPin(0,0) / )

$GCell = PAR(("-$a", "-$b")("-$a", $b)(0, $c) ! 1-3 !
          ( $a , $b)( $a , "-$b")(0, "-$c") ! 4-6 !
          / 6,cool / GPin(0,0) / )

dpin = CCS ( $rdi, $rdo / / deti, deto)

cw = STR ( (0,0) (0,$d2) ($aa,$d2) ($aa,0) ! 1- 4!
          (0,$d1) ($aa,$d1) ! 5- 6!
          ($ac,$d1) / 4,blade / apin ($ac,$d2) 1 /
          1,5,6,4,blade )

apin = CCS ( $rab / / abs)

wing = CNX ( cw,cw,cw,cw / (1,3,4)0(2,2,1) /
          (2,3,4)0(3,2,1) / (3,3,4)0(4,2,1) ) !because k=0!

cc = STR ( (0,0) (0,$clong) ($c2,$clong) !nodes 1- 3!
          ($c2,$c2) ($clong,$c2) ($clong,0) !nodes 4- 6!
          ($c1,0) ($c2,0) ($c3,0) ($c4,0) !nodes 7-10!
          (0,$c1) ($c1,$c1) ($c2,$c1) ($c3,$c1) !nodes 11-14!
          ($c4,$c1) ($clong,$c1) !nodes 15-16!
          (0,$c2) ($c1,$c2) ($c3,$c2) ($c4,$c2) !nodes 18-20!
          (0,$c3) ($c1,$c3) ($c2,$c3) !nodes 21-23!
          (0,$c4) ($c1,$c4) ($c2,$c4) ($c1,$clong) !nodes 24-27!
          / 6,blade / / 1,11,12, 7,blade; 7,12,13, 8,blade;
          8,13,14, 9,blade; 9,14,15,10,blade; 10,15,16, 6,blade;
          11,17,18,12,blade; 12,18, 4,13,blade; 13, 4,19,14,blade;
          14,19,20,15,blade; 15,20, 5,16,blade; 17,21,22,18,blade;
          18,22,23, 4,blade; 21,24,25,22,blade; 22,25,26,23,blade;
          24, 2,27,25,blade ) !Last is default, blade!

$peri = PAR ( (0,0) (0,$p) ($p,$p) ($p,0)
          (0,"$p/2") ("$p/2",$p) ($p,"$p/2") ("$p/2",0) )

$cregs = PAR ( 1,5,9,12,8,cool; 5,2,6,10,9,cool; 6,3,7,11,10,cool )

$cell1 = PAR ( $peri ("$p/2-$rp1","$p/2") ("$p/2","$p/2+$rp1")
          ("$p/2+$rp1","$p/2") ("$p/2","$p/2-$rp1")
          / 4,cool / pin1 ("$p/2","$p/2") / $cregs )

$Fside = PAR(("-$a", "-$b")("-$a", $h3)( $a , $h3)($a, "-$b")(0, "-$c") ! 1-5 !
          ("-$a", $b)( 0 , $h1)( $a , $b ) ! 6-8 !
          ("-$a", $h1)( $a , $h1)("-$a", $h2)($a, $h2) ! 9-12 !
          ("-$d", "-$e")("-$d", $e)( 0 , $f ) ! 13-15 !
          ( $d , $e )( $d , "-$e")( 0 , "-$f") ! 16-18 !
          / 5,shroud/ Fpin(0,0) / 1,6,14,13,cool; 6,7,15,14,cool; ! 1-3 !
          7,8,16,15,cool; 8,4,17,16,cool; 4,5,18,17,cool; ! 4-6 !
          5,1,13,18,cool; 6, 9, 7,cool; 7,10, 8,cool; ! 7-9 !
          11,2,3,12,gap ) ! 10-10 !

$Bside = PAR(("-$a", "-$b")("-$a", $h3)( $a , $h3)($a, "-$b")(0, "-$c") ! 1-5 !
          ("-$a", $b)( 0 , $h1)( $a , $b ) ! 6-8 !
          ("-$a", $h1)( $a , $h1)("-$a", $h2)($a, $h2) ! 9-12 !
          ("-$d", "-$e")("-$d", $e)( 0 , $f ) ! 13-15 !
          ( $d , $e )( $d , "-$e")( 0 , "-$f") ! 16-18 !
          / 5,shroud/ Bpin(0,0) / 1,6,14,13,cool; 6,7,15,14,cool; ! 1-3 !
          7,8,16,15,cool; 8,4,17,16,cool; 4,5,18,17,cool; ! 4-6 !
          5,1,13,18,cool; 6, 9, 7,cool; 7,10, 8,cool; ! 7-9 !
          11,2,3,12,gap ) ! 10-10 !

```



```

$FSide = PAR(("-$a", "-$b")("$a", $h3)( $a, $h3)($a, "-$b")(0, "-$c") ! 1-5 !
          ("-$a", $b)( 0, $h1)( $a, $b ) ! 6-8 !
          ("-$a", $h1)( $a, $h1)("$a", $h2)($a, $h2) ! 9-12 !
          / 5,cool / FPin(0,0) / 6, 9, 7,cool; 7,10, 8,cool; ! 1-3 !
          9,11,12,10,shroud; 11,2,3,12,gap ) ! 4-5 !

$BSide = PAR(("-$a", "-$b")("$a", $h3)( $a, $h3)($a, "-$b")(0, "-$c") ! 1-5 !
          ("-$a", $b)( 0, $h1)( $a, $b ) ! 6-8 !
          ("-$a", $h1)( $a, $h1)("$a", $h2)($a, $h2) ! 9-12 !
          / 5,cool / BPin(0,0) / 6, 9, 7,cool; 7,10, 8,cool; ! 1-3 !
          9,11,12,10,shroud; 11,2,3,12,gap ) ! 4-5 !

$cell2 = PAR ($peri (" $p/2-$rp2", "$p/2") (" $p/2", "$p/2+$rp2")
              (" $p/2+$rp2", "$p/2") (" $p/2", "$p/2-$rp2")
              / 4,cool / pin2 (" $p/2", "$p/2") / $cregs )

$cell3 = PAR ( $peri (" $p/2-$rp3", "$p/2") (" $p/2", "$p/2+$rp3")
              (" $p/2+$rp3", "$p/2") (" $p/2", "$p/2-$rp3")
              / 4,cool / pin3 (" $p/2", "$p/2") / $cregs )

$cell4 = PAR ( $peri (" $p/2-$rp4", "$p/2") (" $p/2", "$p/2+$rp4")
              (" $p/2+$rp4", "$p/2") (" $p/2", "$p/2-$rp4")
              / 4,cool / pin4 (" $p/2", "$p/2") / $cregs )

$row = PAR ( !Coupling of 6 structures, the first and last are the
             wide and narrow gap sides, the others are pin cells.
             Rotate the wide side by -pi/2, the narrow by pi/2!
             (1,4,1)$k(2,2,1) / (2,3,4)$k(3,2,1) / (3,3,4)$k(4,2,1) /
             (4,3,4)$k(5,2,1) / (5,3,4)$k(6,1,4) )

```

```

$x2      = PAR("-( $b+$h3)*3**0.5/2")
$x10     = PAR("-( $b+$h1)*3**0.5/2")
$x13     = PAR("-( $b+$h2)*3**0.5/2")
$y2      = PAR(" ( $b+$h3)/2-$c")
$y10     = PAR(" ( $b+$h1)/2-$c")
$y13     = PAR(" ( $b+$h2)/2-$c")

$x7      = PAR("-$h1*3**0.5/2")
$y7      = PAR(" $h1/2")

$x3      = PAR("3*$p- $AssemblyPitch/(2*3**0.5)")
$x11     = PAR("3*$p-($ShroudOuterDimension-2*$ShroudThickness)/(2*3**0.5)")
$x14     = PAR("3*$p- $ShroudOuterDimension/(2*3**0.5)")
top      = CNX ( ww, wgx, wgx, wgx, wg, wn !Rotate 'ww' by -pi/2!
               / (1,4,1)2(2,2,1) / (2,3,4)0(3,2,1) / (3,3,4)0(4,2,1)
               / (4,3,4)2(5,12,1)/ (5,3,4)2(6,2,1) )

bot      = CNX ( nw, ng, ng, ng, ng, nn !rotate 'nw' & 'ng' by pi; 'nn' by pi/2!
               / (1,1,2)2(2,4,3) / (2,1,2)0(3,4,3) / (3,1,2)0(4,4,3)
               / (4,1,2)0(5,4,3) / (5,1,2)2(6,1,4) )

row1     = CNX ( wgx, $prow1, ng / $row )
row2     = CNX ( wgx, $prow2, ng / $row )
row3     = CNX ( wgx, $prow3, ng / $row )
row4     = CNX ( wg, $prow4, ng / $row )

system   = CNX ( top, row1, row2, row3, row4, bot, cc, wing, wing
               / (1-1,2)(1-6,4)$k(2-1,3)(2-6,2) / (2-1,2)(2-6,3)$k(3-1,3)(3-6,2)
               / (3-1,2)(3-6,3)$k(4-1,3)(4-6,2) / (4-1,2)(4-6,3)$k(5-1,14)(5-6,2)
               / (5-1,2)(5-6,3)$k(6-1,4)(6-6,2) / (1-1,2,4)$k(7,20,26)
               / (7,2,3)$k(8-1,2,1) / (7,5,6)$k(9-4,4,3) )

system   = NEWK( !Adjust corner-gap coupling to k=2!
                2 / (1-1,1,2) (1-6,4,1) (6-1,4,1) (6-6,1,2) /
                !Adjust inter-gap coupling to k=0 (wgx with wg gets k=2)!
                0 / (2-1,1,2) (3-1,1,2) !Left side! /
                2 / (4-1,1,2) !Left side! /
                0 / (2-6,3,4) (3-6,3,4) (4-6,3,4) !Right side! )

system   = BDRY( !Diagonal albedo all around!
                (7,1,1) $k( White ) !From "cc" corner to "cc" corner!

tov1     = OVLT( $tcool/*-**-*/ $tfuel/*-**-fuel )
tos1     = OVST( tov1 )

v40u    = STAT( mos1, dos40, tos1, $pw )
v40c    = STAT( mos2, dos40, tos1, $pw )
v70u    = STAT( mos1, dos70, tos1, $pw )

```

Reactor core geometry

```

$Saxton = PAR( $Layout /
  ( 61,6,1)$k( 73,4,3) / ( 60,6,1)$k( 72,4,3) / ( 59,6,1)$k( 71,5,4) /
  ( 53,4,6)$k( 34,1,6) / ( 96,2,4)$k( 15,1,6) / ( 68,4,6)$k( 16,1,6) /
  ( 74,2,4)$k( 17,1,6) / ( 47,4,6)$k( 18,1,6) / ( 29,2,4)$k( 19,1,6) /
  ( 64,1,2)$k( 63,5,4) / ( 63,1,2)$k( 62,5,4) / ( 62,1,2)$k( 61,5,4) /
  ( 61,1,2)$k( 60,5,4) / ( 60,1,2)$k( 59,5,4) / ( 59,1,2)$k( 58,1,5) /
  ( 10,6,1)$k( 50,1,7) / ( 10,1,2)$k( 65,3,7) / ( 14,2,3)$k( 73,1,7) /
  ( 64,4,5)$k( 65,2,1) / ( 65,4,5)$k( 66,2,1) / ( 66,4,5)$k( 67,2,1) /
  ( 67,4,5)$k( 68,2,1) / ( 68,4,5)$k( 69,2,1) / ( 69,4,5)$k( 70,1,5) /
  ( 64,2,3)$k( 51,6,5) / ( 63,2,3)$k( 50,6,5) / ( 62,2,3)$k( 49,6,5) /
  ( 61,2,3)$k( 48,6,5) / ( 60,2,3)$k( 47,6,5) / ( 59,2,3)$k( 46,1,5) /
  ( 64,3,4)$k( 52,1,6) / ( 65,3,4)$k( 53,1,6) / ( 66,3,4)$k( 54,1,6) /
  ( 67,3,4)$k( 55,1,6) / ( 68,3,4)$k( 56,1,6) / ( 69,3,4)$k( 57,5,4) /

```

(64,6,1)\$k(76,4,3) / (63,6,1)\$k(75,4,3) / (62,6,1)\$k(74,4,3) /
(10,6,1)\$k(50,1,7) / (10,1,2)\$k(65,3,7) / (14,2,3)\$k(73,1,7) /
(61,6,1)\$k(73,4,3) / (60,6,1)\$k(72,4,3) / (59,6,1)\$k(71,5,4) /
(64,5,6)\$k(77,3,2) / (65,5,6)\$k(78,3,2) / (66,5,6)\$k(79,3,2) /
(67,5,6)\$k(80,3,2) / (68,5,6)\$k(81,3,2) / (69,5,6)\$k(82,1,5) /
(51,3,4)\$k(40,1,6) / (51,2,3)\$k(39,6,5) / (50,2,3)\$k(38,6,5) /
(49,2,3)\$k(37,6,5) / (48,2,3)\$k(36,6,5) / (47,2,3)\$k(35,1,5) /
(52,3,4)\$k(41,1,6) / (53,3,4)\$k(42,1,6) / (54,3,4)\$k(43,1,6) /
(10,6,1)\$k(50,1,7) / (10,1,2)\$k(65,3,7) / (14,2,3)\$k(73,1,7) /
(23,4,6)\$k(81,1,6) / (45,2,4)\$k(19,1,6) / (34,4,6)\$k(10,1,6) /
(45,2,4)\$k(23,1,6) / (84,4,6)\$k(12,1,6) / (55,2,4)\$k(13,1,6) /
(61,6,1)\$k(73,4,3) / (60,6,1)\$k(72,4,3) / (59,6,1)\$k(71,5,4) /
(53,4,6)\$k(34,1,6) / (96,2,4)\$k(15,1,6) / (68,4,6)\$k(16,1,6) /
(74,2,4)\$k(17,1,6) / (47,4,6)\$k(18,1,6) / (29,2,4)\$k(19,1,6) /
(88,3,5)\$k(21,1,7) / (32,2,4)\$k(22,1,7) / (10,3,5)\$k(24,1,7) /
(11,2,4)\$k(25,1,7) / (13,3,5)\$k(27,1,7) / (34,2,4)\$k(28,1,7) /
(14,3,5)\$k(30,1,7) / (58,2,4)\$k(31,1,7) / (65,3,5)\$k(33,1,7) /
(17,2,4)\$k(34,1,7) / (34,3,5)\$k(36,1,7) / (19,2,4)\$k(37,1,7) /
(21,1,2)\$k(20,7,6) / (67,1,2)\$k(23,7,6) / (27,1,2)\$k(26,7,6) /
(30,1,2)\$k(29,7,6) / (10,1,2)\$k(32,7,6) / (36,1,2)\$k(35,7,6) /
(20,2,6)\$k(38,1,1) / (20,2,6)\$k(39,1,1) / (26,2,6)\$k(40,1,3) /
(29,2,6)\$k(41,17,13) / (32,2,6)\$k(42,1,3) / (35,2,6)\$k(43,7,3) /
(76,5,6)\$k(88,3,2) / (76,6,1)\$k(87,4,3) / (75,6,1)\$k(86,4,3) /
(74,6,1)\$k(85,4,3) / (73,6,1)\$k(84,4,3) / (72,6,1)\$k(83,5,4) /
(77,5,6)\$k(89,3,2) / (78,5,6)\$k(90,3,2) / (79,5,6)\$k(91,3,2) /
(40,2,3)\$k(29,6,5) / (39,2,3)\$k(28,6,5) / (38,2,3)\$k(27,6,5) /
(40,3,4)\$k(30,1,6) / (41,3,4)\$k(31,1,6) / (42,3,4)\$k(32,1,6) /
(88,6,1)\$k(98,4,3) / (87,6,1)\$k(97,4,3) / (86,6,1)\$k(96,4,3) /
(88,5,6)\$k(99,3,2) / (89,5,6)\$k(100,3,2) / (90,5,6)\$k(101,3,2) /
(29,3,4)\$k(20,1,6) / (29,2,3)\$k(19,6,5) / (28,2,3)\$k(18,6,5) /
(30,3,4)\$k(21,1,6) / (31,3,4)\$k(22,1,6) / (32,3,4)\$k(23,1,6) /
(98,5,6)\$k(108,3,2) / (98,6,1)\$k(107,4,3) / (97,6,1)\$k(106,4,3) /
(99,5,6)\$k(109,3,2) / (100,5,6)\$k(110,3,2) / (101,5,6)\$k(111,3,2) /
(20,2,3)\$k(11,6,5) / (19,2,3)\$k(10,6,5) / (18,2,3)\$k(9,6,5) /
(20,3,4)\$k(12,1,6) / (21,3,4)\$k(13,1,6) / (22,3,4)\$k(14,1,6) /
(108,6,1)\$k(116,4,3) / (107,6,1)\$k(115,4,3) / (106,6,1)\$k(114,4,3) /
(108,5,6)\$k(117,3,2) / (109,5,6)\$k(118,3,2) / (110,5,6)\$k(119,3,2) /
(11,3,4)\$k(4,1,5) / (11,2,3)\$k(3,5,4) / (10,2,3)\$k(2,5,4) /
(12,3,4)\$k(5,1,5) / (13,3,4)\$k(6,1,5) / (14,3,4)\$k(7,1,5) /
(116,5,6)\$k(124,5,4) / (116,6,1)\$k(123,1,5) / (115,6,1)\$k(122,1,5) /
(64,5,6)\$k(77,3,2) / (65,5,6)\$k(78,3,2) / (66,5,6)\$k(79,3,2) /
(67,5,6)\$k(80,3,2) / (68,5,6)\$k(81,3,2) / (69,5,6)\$k(82,1,5) /
(51,3,4)\$k(40,1,6) / (51,2,3)\$k(39,6,5) / (50,2,3)\$k(38,6,5) /
(49,2,3)\$k(37,6,5) / (48,2,3)\$k(36,6,5) / (47,2,3)\$k(35,1,5) /
(117,5,6)\$k(125,5,4) / (118,5,6)\$k(126,5,4) / (119,5,6)\$k(127,1,5) /
(108,6,1)\$k(116,4,3) / (107,6,1)\$k(115,4,3) / (106,6,1)\$k(114,4,3) /
(108,5,6)\$k(117,3,2) / (109,5,6)\$k(118,3,2) / (110,5,6)\$k(119,3,2) /

Burnup and output flow

```
&ADD      = SET ( 'USRC01.hrf' / SAXTON; ZENITH; &v305A )

C1        = IMP(HELIOS;$Hcase/ $Hfile)
Burnup    = SEL(uburn / status/ C1; / $scalp)
$mapsize  = PAR(58;13)

! Compute the power distribution !
fx        = SEL(fx / macro/ C1;Maps/ $scalp)
kf        = SEL(kf / macro/ C1;Maps/ $scalp)
vo        = SEL(vo / macro/ C1;Maps)
power     = FOR(fx*kf*vo)

bu        = SEL(bu / macro/ C1;Maps / $scalp)
rho       = SEL(rho / micro/ C1;FuelIsos/ $scalp)
```

```
ip      = IND(bu / PER:RIEGO)
il      = IND(bu / RAN:1; ; ; )
is      = IND(bu / SOR:)
bl      = FOR(bu^is)
el      = FOR(@nbE(bu)^is)
rl      = FOR(@nbR(bu)^is)
ia      = IND(e1 / SOR:ascen)
'Max burn' = FOR(((bl^ia)^ip)^il)
'at pin'   = FOR(((rl^ia)^ip)^il)
```

```

ip      = IND(power / PER:RIEGO)
il      = IND(power / RAN:1;;; )
is      = IND(power / SOR:)
pl      = FOR(power^is)
el      = FOR(@nbE(power)^is)
r1      = FOR(@nbR(power)^is)
ia      = IND(e1 / SOR:ascen)
'Max W/cm' = FOR((pl^ia)^ip^il)
' at pin' = FOR((r1^ia)^ip^il)

ipos    = IND(power / WIND:1.0E-20,/ 1.0)
rho     = FOR((power*rho)^ipos)^ipos)
powdens = FOR(power/(rho*vo))
ip      = IND(powdens/ PER:RIEGO)
is      = IND(powdens/ SOR:)
pl      = FOR(powdens^is)
el      = FOR(@nbE(powdens)^is)
r1      = FOR(@nbR(powdens)^is)
ia      = IND(e1 / SOR:ascen)
'Max W/gU' = FOR((pl^ia)^ip^il)
' at pin' = FOR((r1^ia)^ip^il)

MaxVals = LIST(Burnup;S/
               'Maximum burnup, power density [W/gU] and linear power [W/cm]';
               'Power peaking factor (ppf) without gamma smearing'
               /E/
               f0:'Max burn'      ; 'at pin' ;
               f3:'Max W/gU'     ;f0: ' at pin';
               f3:'Max W/cm'     ;f0: ' at pin' ;
               f4: ppf          )

&ADD      = SET ( 'USRC01.hrf' / SAXTON; ZENITH; &v305A )

$hiso    = PAR(92235; 92236; 92238; 94238; 94239; 94240; 94241; 94242)
$avo     = PAR(0.6023)
ndens    = SEL(nd / micro/ C1;FuelIsos/ $scalp/ $hiso)
aw       = SEL(aw / micro/ C1;FuelIsos/ $scalp/ $hiso)
rho      = SEL(rho / micro/ C1;FuelIsos/ $scalp)
vol      = SEL(vo / micro/ C1;FuelIsos)

nd       = FOR(@smR(ndens*vol)/@smR(vol))
'g/cm'   = FOR(@smR(vol)*nd*aw/$avo)
'wt%'    = FOR(100 *'g/cm' /@smR(rho*vol))

iu5      = IND(nd /RAN:;1;;; )
iu6      = IND(nd /RAN:;2;;; )
iu8      = IND(nd /RAN:;3;;; )
ip8      = IND(nd /RAN:;4;;; )
ip9      = IND(nd /RAN:;5;;; )
ip0      = IND(nd /RAN:;6;;; )
ip1      = IND(nd /RAN:;7;;; )
ip2      = IND(nd /RAN:;8;;; )

U235     = FOR(nd^iu5)
U236     = FOR(nd^iu6)
U238     = FOR(nd^iu8)
Pu238    = FOR(nd^ip8)
Pu239    = FOR(nd^ip9)
Pu240    = FOR(nd^ip0)
Pu241    = FOR(nd^ip1)
Pu242    = FOR(nd^ip2)

InvNd    = LIST(Burnup;S/
               'Average fuel composition in at/barn-cm'

```

/E/

e5:U235; U236; U238; Pu238; Pu239; Pu240; Pu241; Pu242)

```

U235      = FOR('g/cm'^iu5)
U236      = FOR('g/cm'^iu6)
U238      = FOR('g/cm'^iu8)
Pu238     = FOR('g/cm'^ip8)
Pu239     = FOR('g/cm'^ip9)
Pu240     = FOR('g/cm'^ip0)
Pu241     = FOR('g/cm'^ip1)
Pu242     = FOR('g/cm'^ip2)

InvG      = LIST(Burnup;S/
                'Average fuel composition in g/cm'
                /E/
                f4:U235; U236; U238; Pu238; Pu239; Pu240; Pu241; Pu242)

U235      = FOR('wt%'^iu5)
U236      = FOR('wt%'^iu6)
U238      = FOR('wt%'^iu8)
Pu238     = FOR('wt%'^ip8)
Pu239     = FOR('wt%'^ip9)
Pu240     = FOR('wt%'^ip0)
Pu241     = FOR('wt%'^ip1)
Pu242     = FOR('wt%'^ip2)

InvRel    = LIST(Burnup;S/
                'Average fuel composition in weight % of initial heavy mass'
                /E/
                f4:U235; U236; U238; Pu238; Pu239; Pu240; Pu241; Pu242)

&ADD      = SET ( 'USRC01.hrf' / SAXTON; ZENITH; &v305A )
burnup    = SEL(bu      / macro/ C1;Maps/ $scalp)
cxfuel    = SEL(vx      / macro/ C1;Maps)
cyfuel    = SEL(vy      / macro/ C1;Maps)
nopins    = SEL(nra     / macro/ C1;Maps)

avpow     = FOR(@smR(power)/nopins)
'power peaking' = FOR(power/avpow)
ppf       = FOR(@mxR('power peaking'))

&ADD      = SET ( 'USRC01.hrf' / SAXTON; ZENITH; &v305A )

cx        = SEL(vx      / micro/ C1;FuelIsos)
cy        = SEL(vy      / micro/ C1;FuelIsos)
Volumes   = SEL(vo      / micro/ C1;FuelIsos)
Isotopes  = SEL(id      / micro/ C1;FuelIsos//$Isos)
'N(fuelpin)' = SEL(nd/ micro/ C1;FuelIsos/ $scalp / $Isos)

PinMap    = MAP(;24 /$mapsize /
                'Output for fuel isotopics per pin';
                'Map showing the fuel pin numbers and volumes'
                / cx; cy/ Volumes)

IsoList   = LIST(;S/
                'Fuel isotopics per pin';
                'List of isotope identifiers' /I/
                f0:Isotopes)

NdList    = LIST(Burnup /
                'Fuel isotopics per pin';
                'Number densities in atoms/barn-cm per fuel pin and isotope';
                'R-I-E-.-. indicates fuel pin R and isotope I' /E/
                'N(fuelpin)')

```

10. Acronyms and Abbreviations

AEC	Atomic Energy Commission
BOL	Beginning Of Life
DOE	Department of Energy
EOL	End Of Life
ID	Inner Diameter
In.	Inches (1 inch is 2.54 cm)
MCR	Middle Control Rod
Mils	One thousandth of one inch
MOL	Middle Of Life
MOX	Mixed Oxides or PuO ₂ - UO ₂
MOXP	Mixed Oxides Pelletized
MOXV	Mixed Oxides Vipac
MWD/t	Megawatt Days per metric ton
NRC	Nuclear Regulatory Commission
OD	Outer Diameter
SPPC	Saxton Partial Plutonium Core
SS	Stainless Steel
TD	Theoretical Density
UOX	Uranium Oxide
USRC	Unified Saxton Reactor Core Model
ZR	Zirconium

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